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HIGH ALTITUDE ATMOSPHERIC RADIATION TRANSPORT CALCULATIONS

Kaman Sciences Corporation Colorado Springs, CO 80907

August 1976

Final Report

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Raymond a. Shulstad

RAYMOND A SHULSTAD Captain, USAF Project Officer

FOR THE COMMANDER

Terry M. Lawritsen TERRY N. LAURITSEN

Lt Colonel, USAF

Chief, Technology and Analysis

Branch

PAUL J. DAIL∜ Colonel, USAF

Chief, Analysis Division

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This report contains the results of MORSAIR code Monte Carlo calculations of neutron and secondary gamma transport in a real two-dimensional variable density atmosphere. These data were generated so that an assessment could be made of the adequacy of mass integral scaling of uniform air calculations in defining radiation environments in the atmosphere. Unclassified fission and thermonuclear source spectra were used in the calculations at source altitudes from 5 to 80 kilometers. Silicon and tissue doses as well as the total fluences were calculated for both neutrons and secondary gammas. The $4\pi R^2$ dose ences were calculated for both neutrons and secondary gammas.

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PREFACE

The author wishes to express his appreciation to Capt. Raymond A. Shulstad, the AFWL Project Officer, for his interest and assistance in the generation and evaluation of the data contained in this report, and to Eleanor Berthelot of Kaman Sciences for her assistance in making the MORSAIR runs, processing and fitting the results, and plotting the data.

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SECTION I INTRODUCTION

This report presents the results of fifteen computer runs made with the Kaman Sciences MORSAIR Monte Carlo computer program. These calculations were performed for the Air Force Weapons Laboratory (AFWL/SAT) under the direction of Capt. Raymond A. Shulstad.

The objective of this effort was to provide AFWL with a set of "real" two-dimensional atmospheric neutron and secondary gamma transport data for use in assessing the adequacy of mass scaled uniform air calculations. Previous studies at Kaman (ref. 1) and elsewhere (refs. 2, 3, 4) have indicated that large inaccuracies can result from scaling at high altitudes. The results presented in this report show the extent of these inaccuracies for two unclassified source spectra for both neutron and secondary gamma doses at altitudes from 5 to 80 kilometers.

For each MORSAIR run, the neutron and secondary gamma doses in silicon and tissue as well as the total particle fluence were calculated using a Monte Carlo technique at more than 150 detector locations about a point isotropic source in the atmosphere. The spectra used in these runs were an unclassified fission source and an unclassified thermonuclear source, both of which were provided by AFWL (ref. 5). In addition to a run in homogeneous air with each source spectrum for the purposes of verifying the tracking and scoring techniques of the code, eight runs with the fission source and five with the thermonuclear source were made in the variable density atmosphere. Section II of this report describes in detail the energy bins, spectra, altitudes and dose response functions used in the calculations.

In section III the homogeneous atmosphere neutron and secondary gamma transport data supplied by AFWL and used in this study are described. The two homogeneous atmosphere MORSAIR runs made for checkout purposes are discussed. Also, in section III, the statistical uncertainties of the Monte Carlo data, the method used to fit these data, and the accuracy of these fits are briefly described. Finally, the method used to determine the K-factor, defined as the ratio of the $4\pi R^2$ dose in real variable density air to the $4\pi R^2$ dose in homogeneous air, is described.

In section IV, a brief discussion of some of the more obvious trends of the results is presented. The qualitative dependence of the K-factor on the source and detector altitude, mass range, source spectrum, and dose response function is described. The two primary effects which cause deviations from the scaled homogeneous air results, namely a leakage effect and a mass distribution effect are also described.

A summary of the calculations performed for this study and the conclusions which may be drawn from these results are presented in section V.

In Appendix A, the tabulated fit coefficients of the one-dimensional ANISN $4\pi R^2$ dose data (used in defining the K-factors) are shown for the two source spectra. All of the $4\pi R^2$ doses have been plotted for both neutrons and secondary gammas and these are included in Appendix A. In Appendix B, the coefficients for the fits to the MORSAIR two-dimensional data are shown for each sampling altitude, dose response function, and particle type (neutron or secondary gamma). In Appendix C, plots of the fitted silicon dose data and the silicon K-factors for both neutrons and secondary gammas are shown for all 15 MORSAIR runs.

SECTION II CALCULATIONS

1. THE MORSAIR PROGRAM

The Kaman Sciences MORSAIR program is an extensively modified version of the ORNL MORSE program (ref. 6) developed specifically for Monte Carlo calculations of radiation environments in a variable density atmosphere based on the 1962 Standard Atmosphere model (ref. 7). The multigroup cross section module of MORSE was used without modification, but the geometry and random walk modules were revised and a new scoring routine using an "extended path" or expectation boundary crossing estimator for scoring in concentric annular rings was added. In previous studies, the MORSAIR code has been used for calculating radiation environments at source altitudes from 20 to 65 kilometers. The results of those calculations have been checked against other Kaman real air transport codes, and codes from other agencies (ref. 8). has been found to be a useful code for predicting radiation environments .t high altitudes where traditional methods of scaling uniform atmosphere results are inadequate.

The latest version of MORSAIR was written for use on the CDC 7600 computer and utilizes the fast access large core capability of that machine so that very large problems in terms of storage requirements can be efficiently handled by the code. In this study the time integrated neutron and secondary gamma doses and associated standard deviations were calculated for 58 energy groups at as many as 190 detectors spaced about the source. The differential energy spectra were recorded on magnetic tape so that doses other than those shown in this report can be calculated.

Earth curvature effects were not included in the calculations, because these effects have been shown to be negligible at the altitudes and ranges of the calculations performed for this study (ref. 9).

2. INPUT PARAMETERS

The air cross sections used in the MORSAIR runs were the DLC-31 multigroup set distributed by the Radiation Shielding Information Center at ORNL (ref. 10). These cross sections were prepared from ENDF/B-IV data using a fission spectrum weighting function. A third order (P3) Legendre expansion was used to represent the angular variation of the cross sections. The nitrogen and oxygen cross sections were mixed to form the macroscopic cross sections of air with the following composition (ref. 5):

Density: 1.11 mg/cm³

Volume percentages: 79 percent nitrogen

21 percent oxygen

Number densities: 3.6609×10^{19} nitrogen atoms/cm³

9.7316 x 10^{18} oxygen atoms/cm³

A copy of these cross sections was provided to Kaman by AFWL. The 37 neutron and 21 secondary gamma energy groups of the DLC-31 cross section set are shown in table 1. In table 2, the fraction of source neutrons in each of the groups is shown for the two unclassified spectra used in the calculations.

Three different response functions were used to weight the MORSAIR energy spectrum at each detector. Table 3 shows the neutron silicon and tissue dose response functions, and table 4 shows these response functions for the secondary gamma groups. The response functions shown in tables 3 and 4 were provided by AFWL (ref. 5). In addition to the silicon and tissue doses, the total number fluence was also calculated for both neutrons and secondary gammas.

TABLE 1
DLC-31 ENERGY GROUP STRUCTURE

Č BONB	NEUTRON ENERGY (MEV)	GROUP	GAMMA ENERGY	(YEV)
1	1.9640E+01 - 1.5905E+01	38	1.4000E+01 -	1.0000E+01
2	1.6905E+01 - 1.4918E+01		1.0000E+01 -	
3	1.4918E+01 - 1.41916+01		8.0000E+00 -	
4	1.4191E+01 - 1.3840E+01		7.0000E+00 -	
5	1.3840E+01 - 1.2840E+01		6.0000E+00 -	
6	1.2948E+01 - 1.2214E+01		5.0000E+00 -	
7	1.22146+01 - 1.105/26+01		4.0000E+00 -	
8	1.1052E+01 - 1.0000E+01	45	3.000E+00 -	
9	1.0000E+31 - 9.3484E+00		2.5000E+00 -	
10	9.0484E+30 - 8.1873E+00	47	2.0000E+00 -	1.5000E+00
11	8.1873E+00 - 7.4082E+00	48	1.5000E+00 -	1.0000E+00
1?	7.4082E+00 - 6.3763E+00	49	1.0000E+00 -	7.0009E-01
13	6.3763E+00 - 4.9659E+00	50	7.0030E-01 -	4.5000E-01
14	4.9659E+00 - 4.7237E+80	51	4.5000E-01 -	3.0000E-01
15	4.7237E+00 - 4.0657F+00	52	3.0000E-01 -	1.5000E-01
16	4.0657E+03 - 3.0119E+0.0	53	1.5000E-01 -	1.0000E-01
17	3.0119E+00 - 2.3852E+00		1.0000E-01 -	7.0000E-02
19	2.3852E+00 - 2.3069E+00		7.0000E-02 -	4.5000E-02
19	2.3069E+00 - 1.3265E+00		4.5000E-02 -	
20	1.8258E+00 - 1.1080E+00		3.0000E-02 -	2.0000E-02
21	1.1080E+00 - 5.5023E-01		2.0000E-02 -	1.0000E-02
22	5.5023E-01 - 1.5764E-01			
23	1.5764E-01 - 1.1109E-01			
24	1.1109E-01 - 5.2475E-02			
25	5.2475E-J2 - 2.4788E-02			
26	2.4788E-02 - 2.1875E-02			
27	2.1875E-02 - 1.0333E-02	-		
28	1.0333E-02 - 3.35+6E-03			
29	3.3546E-03 - 1.2341E-03			
30	1.2341E-03 - 5.8294E-04			
31	5.8294E-04 - 1.0130E-04			
32	1.0130E-04 - 2.9023E-05			
33	2.9023E-05 - 1.0677E-05			
34	1.0677E-05 - 3.0530E-06			
35	3.0590E-06 - 1.1254E-06			
36	1.1254E-05 - 4.1400E-07			
37	4.1400E-97 - 1.0000E-11	Į.		

TABLE 2 NEUTRON SOURCES

GROUP	NEUTRON ENERGY (MEV)	THERMONUCLEAR	FISSION
1	1.9640E+01 - 1.6905E+01	3.	0.
2	1.6905E+01 - 1.4918E+01	U •	0.
3	1.4918E+01 - 1.4191E+01	1.8870E-02	0.
4	1.41916+01 - 1.38406+01	9.3400E-03	0.
5	1.3840E+01 - 1.2840E+01	2.6620E-02	0 •
6	1.2840E+01 - 1.2214E+01	1.6660E-02	0.
7	1.2214E+01 - 1.1052E+01	1.6870E-J2	0.
8	1.1052E+01 - 1.3000E+01	1.2400E-02	0.
9	1.0000E+01 - 9.0454E+00	7.4800E-03	3.8400E-03
10	9.0484E+00 - 3.1873E+00	6.82002-33	3.5000E-03
11	8.1473E+00 - 7.4082E+00	6.7800E-03	5.3900E-03
12	7.4032E+00 - 6.3763E+00	1.0300E-02	7.3500E-03
13	6.3763E+00 - 4.9659E+00	1.3070E-02	1.8370E-02
14	4.3659E+00 - 4.7237E+00	3.6200E-03	3.2500E-03
15	4.7237E+00 - 4.0657E+00	1.2430E-02	8.4700E-03
16	4.0657E+00 - 3.0119E+00	2.6040E-02	5.5000E-02
17	3.0119E+00 - 2.3852E+00	2.37305-02	3.2440E-02
18	2.3852E+00 + 2.3069E+00	3.7500E-03	1.0580E-02
19	2.3069E+00 - 1.3268E+00	2.5640E-02	9.7240E-02
20	1.8268E+00 - 1.1080E+00	6.4450E-02	1.4677E-01
21	1.1080E+00 - 5.5023E-01	8.8490E-02	2.1567E-01
22	5.5023E-01 - 1.5764E-01	9.1380E-02	1.5018E-01
23	1.5764E-01 - 1.1109E-01	1.1630E-02	1.9300E-02
24	1.1109E-01 - 5.2475E-02	1.1078E-01	1.2098E-01
25	5.2475E-02 - 2.4738E-02	5.4000E-02	5.7290E-02
26	2.4788E-02 - 2.1875E-02	5.6800E-03	ċ.0000E-03
27	2.1875E-02 - 1.0333E-02	9.2640E-02	2.4000E-02
28	1.0333E-02 - 3.3546E-03	1.1627E-01	1.4400E-02
29	3.3546E-03 - 1.2341E-03	7.3820E-02	0.
3 Ú	1.2341E-03 - 5.8294E-04	2.3240E-02	0 •
31	5.8294E-04 - 1.0130E-04	2.0280E-02	0.
32	1.0130E-04 - 2.9023E-05	1.9000E-03	0.
33	2.9023E-J5 - 1.0677E-05	0 •	0.
34	1.0677E-05 - 3.0530E-06	0 •	0.
35	3.0598E-05 - 1.1254E-06	0 •	0.
36	1.1254E-05 - 4.1430E-07	0.	0.
37	4.1400E-07 - 1.0000E-11	9.	0 •

TABLE 3
NEUTRON DOSE RESPONSE FUNCTIONS

GROUP	NEUTRON ENE	RGY (MEV)	TISSUE DOSE	SILICON DOSE
	A 06 00 C 40 A	4 *0055104	RAD/ (N/CM2)	RAD/(N/CM2)
1	1.9648E+01	- 1.5905E+01	8.6724E-09	1.9106E-09
2 3	1.6905E+01	- 1.4918E+01	7.4190E-09	1.7792E-09
	1.4913E+01	- 1.4191E+01	6.8115E-09	1.681 3E-09
4	1.4191£+31	- 1.3840E+01	6.5447E-09	1.6231E-09
5	1.3840E+91	- · - · -	6.1473E-09	1.5144E-09
6	1.28476+01		5.9548E-09	1.3851E-09
7		- 1.1052E+01	5.8936E-09	1.237 0E-09
8	1.1052E+31	- 1.0000E+01	5.5508E-09	1.0530E-09
9	1.0000E+J1	- 3.0494E+00	5.2882E-09	8.7897E-10
10	9.0484E+00	- 8.1873E+00	5.04732-09	7.9629E-10
11	4.1473E+00	- 7.4082E+00	5.0045E-09	7.8141E-10
12	7.4082E+00	- 6.3763E+00	4.7595E-09	4.7092E-10
13	6.3763E+00	- 4.9659E+00	4.4831E-09	2.1394E-10
14	4.9659E+J0	- 4.7237E+00	4.2531E-09	1.8267E-10
15	4.72375+00	- +.0657E+00	4.1711E-03	1.4195E-10
10	4.0057E+30	- 3.0119E+00	3.9784E-09	1.0582E-10
17	3.0119E+00	- 2.3852E+00	3.3905E-09	1.0006E-18
19	2.38525+00	- 2.3069E+00	3.1377E-99	8.2995E-11
19	2.30595+00	- 1.8268E+00	3.0345E-09	9.4778E-11
20	1.8265E+JD	- 1.1050E+00	2.6393E-09	6.5328E-11
21	1.1080E+00	- 5.5023E-01	2.0570E-09	4.9785E-11
22	5.5023E-11	- 1.5764E-01	1.3330E-09	3.1515E-11
23	1.57546-01	- 1.1109E-01	7.62286-10	1.7897E-12
24	1.1109F-01	- 5.2475E-02	5.4890E-10	2.8022E-12
25	5.2475E-02	- 2.4788E-02	3.1164E-10	1.2327E-12
26	2.47885-32	- 2.1875E-02	2.0739E-10	7.9084E-13
27	2.1875E-02	- 1.0333E-02	1.4662E-10	5.8930E-13
28	1.0333E-02	- 3.3546E-03	6.6143E-11	2.9804E-13
24	3.35465-03	- 1.2341E-03	2.2758E-11	1.0498E-13
30	1.2341E-33	- 5.8294E-04	9.13152-12	4.3305E-14
31	5.8294E-04	- 1.0130E-04	3.6632E-12	1.4421E-14
32	1.0133E-04	- 2.3023E-05	1.1759E-12	4.5895E-15
3.5	2.9023E-05	- 1.0677E-05	1.1095E-12	3.9377E-15
34	1.06778-05	- 3.0590E-06	1.6117E-12	5.6286E-15
35	3.0590E-06	- 1.1254E-06	2.7416E-12	9.4023E-15
36	1.1254E-05	- 4.1430E-87	4.4570E-12	1.5390E-14
37	4.1400E-07	- 1.0000E-11	1.1238E-11	7.4244E-13

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TABLE 4
GAMMA DOSE RESPONSE FUNCTIONS

GROUP	GAMMA ENERGY	(MEV)	TISSUE DOSE	SILICON DOSE
			RAD/(N/CM2)	RAD/(N/CM2)
1	1.4000E+01 -	1.0000E+01	2.7431E-09	3.4184E-09
2	1.0000E+01 -	8.J000E+00	2.2564E-09	2.5712E-09
3	8.0000E+00 -	7.0000E+00	1.9840E-09	2.1612E-09
4	7.000GE+00 -	6.0000E+00	1.7922E-09	1.8991E-09
5	6.0000E+00 -	5.0000E+00	1.5928E-09	1.6367E-09
6	5.0000E+00 -	4.0000E+00	1.3897E-19	1.3835E-09
7	4.9000E+90 -	3.30006+00	1.1803E-09	1.1335E-09
8	3.0000E+00 -	2.50386+80	1.0098E-39	9.4334E-10
4	2.500NE+00 -	2.0000E+00	8.8320E-10	8.2034E-10
10	2.0000E+80 -	1.5000E+00	7.4281E-10	6.9343E-10
11	1.500055+00 -	1.0000E+00	5.8030E-10	5.2846E-10
12	1.0000E+00 -	7.9090E-01	4.2393E-10	3.8505E-10
13	7.0000E-01 -	4.5000E-01	2.9695E-10	2.7122E-10
14	4.5000E-01 -	3.0000E-01	1.9283E-10	1.7772E-10
15	3.000005-01 -	1.5000E-01	1.0770E-10	1.0459E-10
16	1.500VE-01 4	1.9000E-01	4.9383E-11	6.851UE-11
17	1.0003E-91 -	7.0030E-02	3.4315E-11	7.9854E-11
18	7.0000E-02 -	4.5000E-02	2.9479E-11	1.4543E-10
19	4.50098-02 -	3.0000E-02	4.3750E-11	3.4411E-10
20	3.000.5-02 -	2.30005-02	J.6647E-11	8.2679E-10
21	2.0000E-02 -	1.9000E-02	3.2504E-10	2.6493E-09

3. DESCRIPTION OF RUNS

A summary of the 15 MORSAIR runs made for this study is shown in table 5. In addition to the homogeneous atmosphere runs, eight runs were made using the fission source in real air at altitudes from 5 to 80 kilometers, and five thermonuclear source runs were made at altitudes from 20 to 80 kilometers. A minimum of 50,000 initial neutron histories were followed for each run, and the secondary gamma production rate was adjusted so that approximately the same number of gammas was produced in each run. For the lower altitude and homogeneous runs, the computer time required for 50,000 histories was greater than 30 minutes of CDC 7600 execution time. At the higher altitudes an appreciable number of the neutrons and secondary gammas escape out the "top" of the atmosphere after a few scatterings so that as many as 150,000 initial neutrons and approximately the same number of secondary gammas could be followed in less than 30 minutes.

A time cutoff of 20 seconds was used for each neutron history. This ensured that almost no neutrons were lost because of this cutoff.

The atmosphere model used in MORSAIR provides for a continuously varying density from sea level to 200 kilometers and very closely approximates the 1962 Standard Atmosphere model (ref. 7). Above 200 kilometers, a void is assumed and particles reaching this altitude are allowed to escape. The lateral extent of the atmosphere varied with each run to provide a large buffer beyond the last detectors of interest at each sampling altitude. This ensured that lateral leakage effects on the results would be negligible. Any particle reaching the ground (which only a very few did) was terminated.

TABLE 5 MORSAIR RUNS

Number	Source	Source Altitude (km)	Number of Detectors	Neutron Histories	CDC 7600 Computer Time (Minutes)
1	Fission	Homogeneous	190	50000	39
7	Fission	ហ	165	50000	42
m	Fission	10	172	50000	36
4	Fission	15	171	50000	33
S	Fission	20	151	50000	31
9	Fission	30	156	00096	30
7	Fission	40	152	150000	30
ω	Fission	09	184	150000	20
0	Fission	80	183	150000	21
10	Thermonuclear	Homogeneous	190	50000	37
11	Thermonuclear	20	151	50000	29
12	Thermonuclear	30	156	93500	30
13	Thermonuclear	40	152	150000	30
14	Thermonuclear	09	184	150000	23
15	Thermonuclear	80	183	150000	20

Particle histories were terminated primarily because they escaped from the top of the atmosphere or their weight became so small that they could not contribute a significant score at any detector.

SECTION III RESULTS

1. HOMOGENEOUS ONE-DIMENSIONAL TRANSPORT DATA

The one-dimensional homogeneous air neutron and secondary gamma transport data used as a basis of comparison in this report were provided by AFWL (ref. 12). These data were generated by appropriate source spectrum weighting of Murphy's fits to Burgio's ANISN results (ref. 11). The resulting $4\pi R^2$ dose data were then fit to the equation:

$$\ln \left[4\pi R^2 \text{ Dose}\right] = A + Bx + Cx^2 + Dx^{3/2} + Ex^{1/2} + Fx^{1/3} + G \ln x$$

for 0.1 < x < 300 gm/cm²

where x is the mass range* of air or areal density in gm/cm². The seven coefficients representing the fits to the neutron and secondary gamma homogeneous atmosphere data are tabulated in Appendix A for the two spectra and three dose response functions used in this study.

For very small areal densities, between 0.0 and 0.1 gm/cm^2 of air which are of interest particularly at very high altitudes, a linear interpolation between the fitted ANISN $4\pi R^2$ dose at 0.1 gm/cm^2 and the uncollided $4\pi R^2$ dose (i.e., the sum over all energy groups of the products of the source fractions and dose response functions) at zero mass range was used to find the $4\pi R^2$ neutron doses. This method is commonly used in mass integral scaling codes to define atmospheric neutron environments at small mass ranges. A similar interpolation technique was used for the secondary gammas between zero and 0.1 gm/cm^2 of air, except in this case a $4\pi R^2$ dose value of zero was used at zero mass range.

^{*}See footnote on page 35 for a definition of mass range.

The same energy groups, cross sections, sources, and dose response functions described earlier for the MORSAIR runs were used in the uniform air calculations.

2. MORSAIR VERIFICATION IN HOMOGENEOUS AIR

An option exists in the MORSAIR code to replace the variable density atmosphere model with a homogeneous atmosphere model. As an initial check on the scoring and tracking techniques employed in the code, one run for both the thermonuclear and the fission source spectrum was made using this It was anticipated that the results of such a run would agree with the one-dimensional ANISN infinite air results provided by AFWL from Burgio's calculations using the same spectra and response functions. Figures C-1 and C-3 of Appendix C show that for all sampling altitudes the difference between the fits of the ANISN homogeneous data and fits of the two-dimensional MORSAIR homogeneous data is less than 15 percent out to 100 gm/cm² and within 25 percent out to 200 gm/cm² for a fission source. Similar results for a thermonuclear source are shown in figures C-49 and C-51 of Appendix C. It should be emphasized that the solid lines shown in these and other figures of Appendix C represent the fitted one-dimensional homogeneous air ANISN d. 1, and the points are calculated from the fits of the MORSAIR data. It should also be pointed out that in the figures immediately following those mentioned above, namely figures C-2, C-4, C-50, and C-52 of Appendix C, the K-factors presented are actually ratios of fits of the homogeneous MORSAIR data to the ANISN data. All of the data in Appendix C is presented in this manner, i.e., the fitted homogeneous and real air data are shown first followed by the K-factors calculated from this data for each MORSAIR The differences between the ANISN and MORSAIR results for a homogeneous atmosphere are within the statistical

uncertainties of the MORSAIR data for these runs, and it was concluded that the MORSAIR tracking and scoring methods were correct.

3. STATISTICAL UNCERTAINTIES IN THE MORSAIR DATA

At the lowest source altitudes, the 50,000 neutron and secondary gamma histories resulted in standard deviations in the total doses of less than 15 percent out to 100 gm/cm², and less than 25 percent to 200 gm/cm² in most cases. At source altitudes above 20 kilometers, the expectation boundary crossing scoring method is more efficient, and the statistical uncertainties in the data were somewhat less.

4. FITS TO THE MORSAIR DATA

To smooth the MORSAIR Monte Carlo results and to reduce the quantity of data to a more usable form consistent with the one-dimensional results, all of the $4\pi R^2$ doses were fit to the same seven parameter equation in areal density described above. For the MORSAIR data, it was found that dropping the last term, G ln x, did not affect the accuracy of the fits significantly and in some cases resulted in a slight improvement in the fit. For this reason, only six coefficients are used in the fits to the MORSAIR data shown in Appendix B.

The technique used to fit the data was a standard linear weighted least squares technique such as described in reference 13. The fitting method, it should be noted, was applied to the logarithm of the $4\pi R^2$ dose data where each data point was given a weight inversely proportional to the square of the logarithmic uncertainty in the MORSAIR data point. Any data point with a standard deviation greater than 50 percent was deleted before the fit was applied. The number of points removed for this reason was small for areal densities less than 200 gm/cm².

The fit coefficients for each run are tabulated in Appendix B. In addition, the range (in terms of minimum and maximum areal densities) over which the fit applies is shown. The last column in the tables, "RMS PCT DIFF", is the weighted root mean square percentage difference between the fit data and the actual MORSAIR data, i.e.,

"RMS PCT DIFF" =
$$\sqrt{\frac{\sum_{i} W_{i} Y_{i}^{2}}{\sum_{i} W_{i}}} \times 100$$

where
$$Y_i = \ln \left[\frac{4\pi R^2 \text{ actual MORSAIR dose}}{4\pi R^2 \text{ fit MORSAIR dose}} \right]$$

and the weight, W_{i} , was defined as:

$$W_{i} = \left[\ln \left(1 + \sigma_{i} \right) \right]^{-2}$$

where σ_i = fractional standard deviation of the MORSAIR dose.

As an example of the adequacy of the six parameter fit to represent the data, figure 1 shows both the actual MORSAIR data points and the values calculated from the fit for coaltitude neutron silicon doses from a thermonuclear source at 20 and 40 kilometers. Figure 2 shows the MORSAIR data points and the fit for the secondary gamma silicon dose. In general, it was found that this fitting technique resulted in a reasonably good representation of the actual data.

5. THE K-FACTOR

The adequacy of mass integral scaling of uniform air results to define real air environments can be conveniently described in terms of a K-factor which was first used by

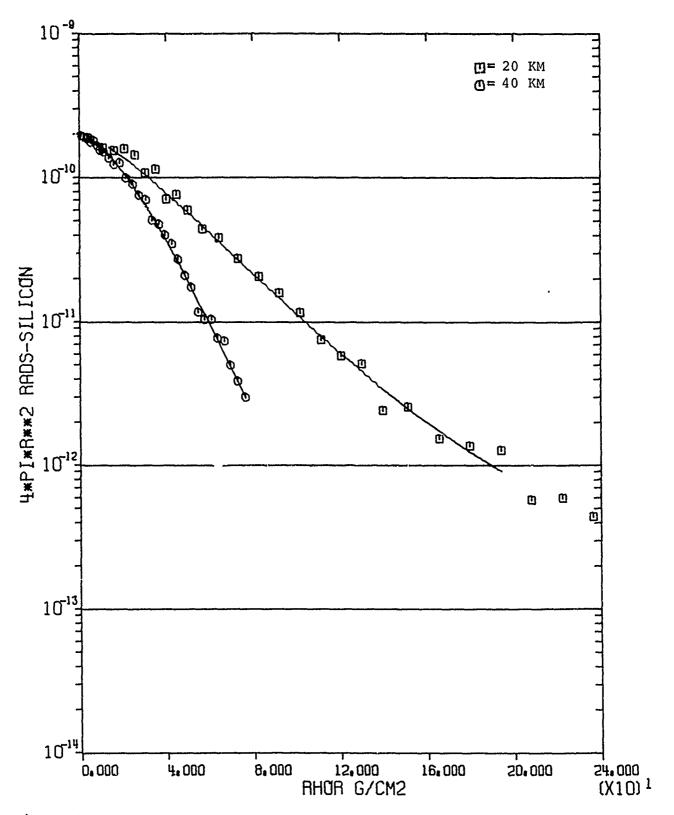


Figure 1. MORSAIR Data Points and the Fit to the Data Points for $4\pi R^2$ Neutron Silicon Dose, Coaltitude Sampling from 20 and 40 km Thermonuclear Sources

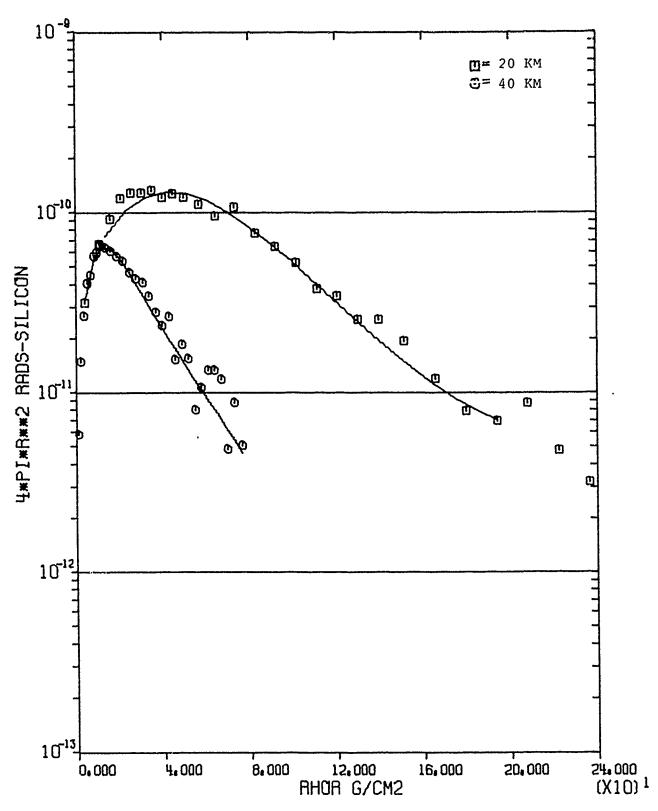


Figure 2. MORSAIR Data Points and the Fit to the Data Points for $4\pi R^2$ Secondary Gamma Silicon Dose, Coaltitude Sampling from 20 and 40 km Thermonuclear Sources

Marcum in his early high altitude transport studies (refs. 2, 3). The K-factor as used in this report is defined as

The $4\pi R^2$ dose at a detector as calculated in a "real" variable density atmosphere (from fits to MORSAIR data in this case)

The $4\pi R^2$ dose received at the detector as calculated in homogeneous one-dimensional infinite air (from fits to the ANISN data in this case)

The K-factor, then, is a direct measure of the error associated with mass integral scaling of infinite air data due to the variable density nature of the atmosphere and can be used as multiplicative correction factor to the scaled data. A K-factor of 0.5, for example, indicates that in the real atmosphere the dose received is only one half of the dose that would be predicted by scaling methods.

Marcum's early studies and more recently those of Keith (ref. 1) and others (refs. 14, 15) have shown that the neutron K-factor is a function of source altitude, detector altitude, slant range, source energy, and the dose response function. The neutron K-factor, it has been shown, can vary from less than .1 at high source altitudes to greater than 5 for detectors below the source and at large areal densities. It is believed that while this study is more extensive in terms of the number of calculations performed and the range of altitudes and mass ranges covered, the data trends shown in this report for neutrons are in substantial agreement with the earlier data of Marcum and Keith. No previous attempts to calculate the transport of secondary gammas in a real atmosphere have been published.

Although there are many situations of practical interest in which the K-factor is near unity so that scaling results in small errors, there are also situations in which large errors can result from neglecting variable density effects. It is hoped that the data presented in this report will allow a user to recognize those situations where scaling may be inaccurate.

In Appendix C, the $4\pi R^2$ silicon dose and the silicon K-factor are plotted as a function of areal density for each of the MORSAIR runs. The trends of the silicon dose K-factors, as will be shown in section IV, are similar to those of the tissue dose and the total fluence K-factors so that in the interest of brevity, only the silicon dose and K-factor plots are shown.

Definition of Mass Range (RHOR):

In homogeneous air,

 $RHOR = \rho(zs)R$

and in 2-D real air,

$$RHOR = \int_0^R \rho(z) dR$$

where

 $\rho(z)$ = density as a function of altitude;

 $\rho(zs)$ = density at source altitude (zs);

R = slant range between the source and a receiver point.

SECTION IV

The K-factor, as defined previously, can be considered as a multiplicative correction factor to be applied to the scaled homogeneous air dose to account for the transport of neutrons and secondary gammas in the real variable density atmosphere. Since many computer codes used to predict neutron and secondary gamma environments in the atmosphere (such as ATR and SMAUG) use the method of mass integral scaling of homogeneous air transport data, it is believed that presenting the MORSAIR results in terms of K-factors allows a direct means of assessing the accuracy of these codes for source spectra similar to those used in this study.

The results of this study show that the K-factor is a complicated function of several parameters. In this section the dependence of the K-factor and therefore the validity of mass integral scaling of homogeneous air data on these parameters is described in a qualitative manner. A detailed explanation of the differences in the transport of neutrons and secondary gammas in homogeneous air and in a variable density atmosphere was beyond the scope of this effort.

1. K-FACTOR DEPENDENCE ON ALTITUDE AND MASS RANGE

In general as the source or detector altitude increases, more particles escape out the top of the atmosphere. Therefore, when compared to the mass integral scaled dose, the dose in a real atmosphere can be substantially less. This leakage effect has been demonstrated in many high altitude transport calculations. Figure 3 shows the coaltitude neutron K-factors for silicon at several source altitudes as calculated by MORSAIR for a fission source. For a detector at 40 gm/cm² from the source, the neutron K-factor is nearly unity for sources below

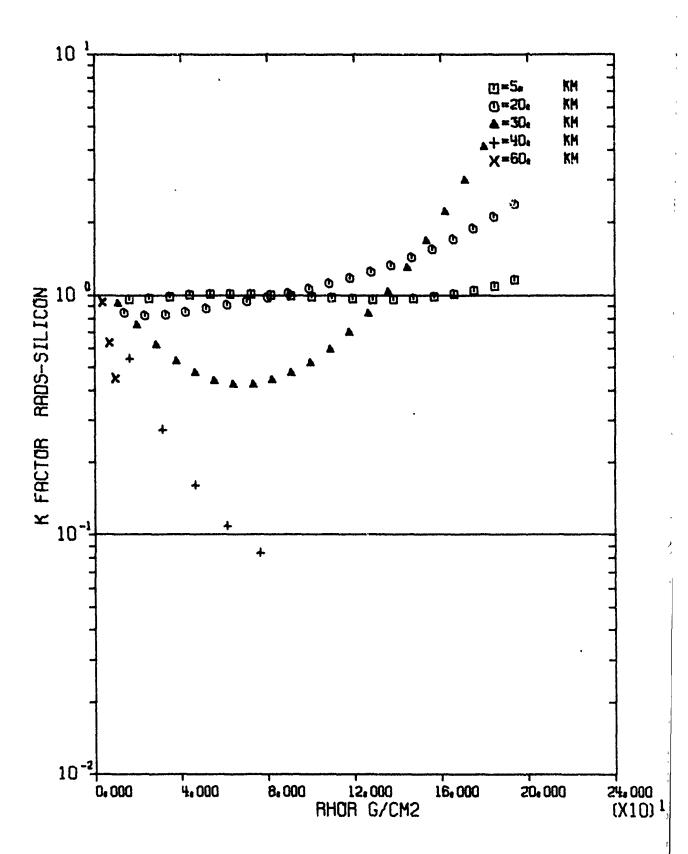


Figure 3. Neutron Coaltitude Silicon K-factors at Several Source Altitudes for a Fission Source

20 kilometers, decreases to about 0.5 for a source at 30 kilometers, and is less than 0.2 for a 40 kilometer source altitude. The secondary gamma silicon K-factors exhibit a similar behavior as shown in figure 4, except the K-factors fall off even more at the higher source altitudes. Similar trends in the K-factors occur if the source altitude remains constant and the detector altitude changes as shown in figures C-27 and C-29 of Appendix C, for example.

In addition to this expected decrease with source and detector altitude due to leakage effects, it is also obvious from figures 3 and 4 that the K-factors are a strong function of the amount of air (areal density) between the source and detector. For detectors coaltitude with the source, the K-factor at high altitudes decreases monotonically as the areal density increases, but at the 20 and .0 kilometer source altitudes, the K-factor curve is more complicated.

Figure 3 shows that for these source altitudes, the neutron K-factor initially decreases with mass range as it does at higher altitudes due to leakage of particles from the atmosphere. At mass ranges of 80 to 100 gm/cm2, however, the neutron K-factor begins to increase and at very large mass ranges it can be greater than 4 indicating that at these large ranges the actual dose can be more than four times the dose that is predicted by scaling the homogeneous transport This dose enhancement has been observed in earlier results (ref. 1), and is apparently due to the fact that a substantial number of the neutrons which arrive at these detectors at large mass ranges travel large distances in the lower density air above the detector and thus traverse less air than is predicted by the simple straight line mass integral between the source and the detector. They are therefore attenuated less than what would be predicted by the scaled

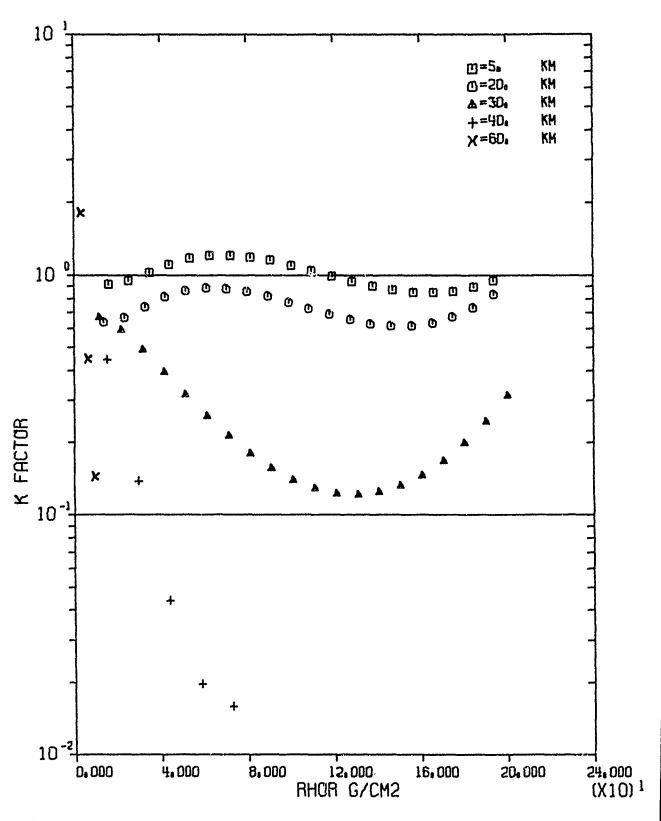


Figure 4. Secondary Gamma Coaltitude Silicon K-factors at Several Source Altitudes for a Fission Source

homogeneous atmosphere results. This effect has been referred to as a "short circuiting" (ref. 15) or "mass distribution" (ref. 14) effect. While it is most apparent for detectors coaltitude and below the source at large mass ranges, this effect also occurs to a lesser extent for detectors above the source at large distances as is apparent from figure C-18, for example.

2. K-FACTOR DEPENDENCE ON DOSE RESPONSE FUNCTIONS

The reduction or enhancement of the neutron dose in the real variable density atmosphere over the mass integral scaled dose depends not only on the source and detector altitudes as shown previously, but also on the dose response function used. For a source at 20 kilometers, it is shown in figure 5 that the neutron K-factors for the silicon and tissue doses and the total fluence are similar for sampling altitudes coaltitude with the source. At 40 kilometers, however, figure 5 shows that the neutron K-factors can be quite different, with the tissue dose K-factor being only half as large as the silicon K-factor, and the K-factor for fluence even smaller. These differences in the K-factors at high altitude are related to the fact that the response functions are different and the energy spectrum arriving at a detector in the real atmosphere can be quite different from the spectrum calculated in homogeneous air at the same areal density. From this example it is clear that the neutron real air effects depend strongly on the dose response function used to weight the neutron fluence.

While the secondary gamma K-factors are strong functions of source altitude and mass range from the source as shown in

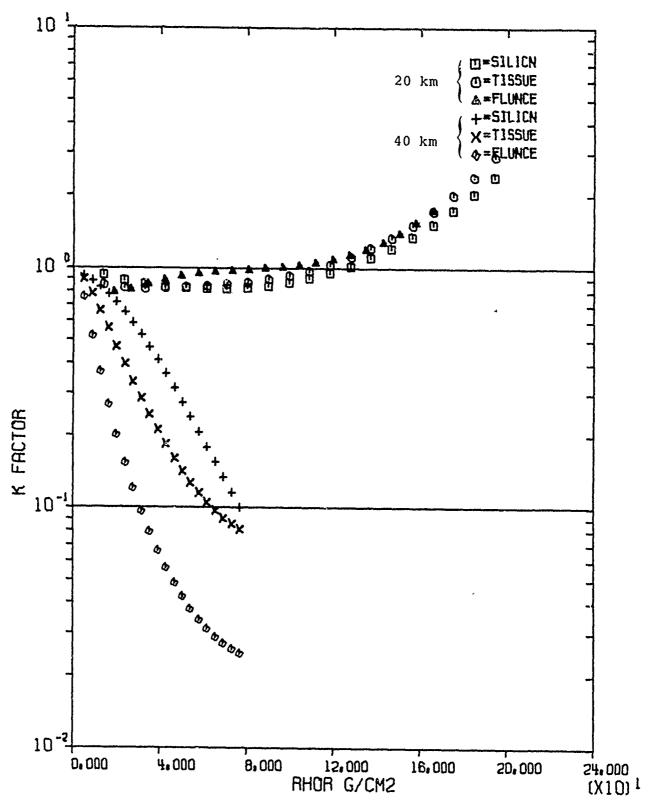


Figure 5. Silicon, Tissue, and Fluence Neutron K-factors Coaltitude from a 20 and 40 km. Thermonuclear Source

figure 6, they are not so dependent on the response functions as was the case for neutrons.

3. K-FACTOR DEPENDENCE ON SOURCE SPECTRUM

In figure 7 the coaltitude neutron silicon dose K-factor is shown for both the thermonuclear and fission source spectra at source altitudes of 20 and 40 kilometers. At 20 kilometers, the differences between the two K-factors are small at all areal densities, despite the fact that the two initial emitted spectra are quite different. At 40 kilometers, however, the silicon dose K-factors for the thermonuclear source are much larger than the K-factors for the fission source at areal densities less than 70 gm/cm². At 40 gm/cm², for example, the coaltitude neutron silicon K-factor for a thermonuclear source spectrum is twice as large as the Kfactor for a fission source. Similar results for the secondary gamma doses are shown in figure 8. At 60 qm/cm² the silicon K-factor for secondary gammas from a thermonuclear source is less than one-third of the silicon dose K-factor from a fission source spectrum coaltitude from a source at 40 kilometers. At 20 kilometers, however, the secondary gamma K-factors are nearly identical.

It is apparent from figures 7 and 8 that mass integral scaling can lead to very large errors in the neutron and secondary gamma environments in the atmosphere, and that the magnitude of these errors depends rather strongly on the source spectrum. In general, the K-factors for the fission source are less than those for a thermonuclear source indicating the scaling errors for this source spectrum are even greater than they are for a thermonuclear source.

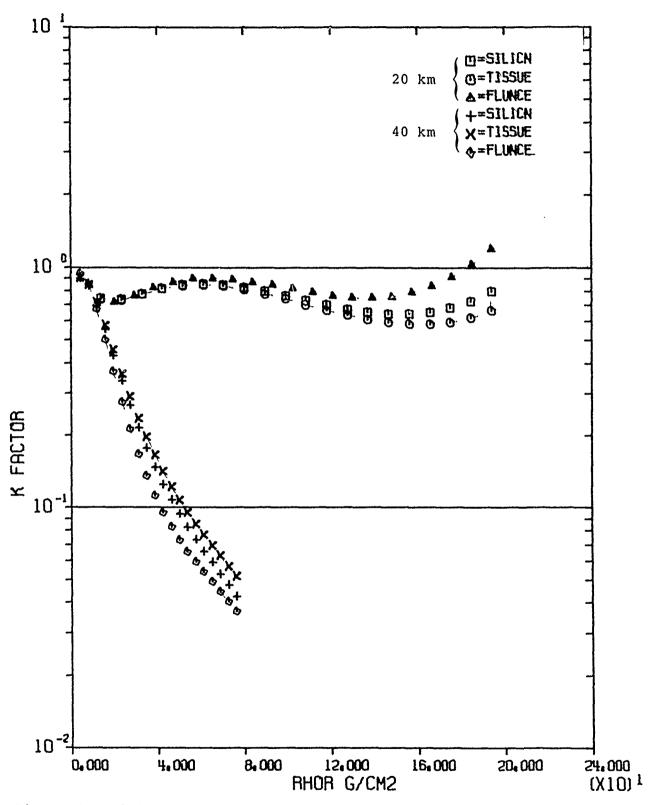


Figure 6. Silicon, Tissue, and Fluence Secondary Gamma K-factors Coaltitude from a 20 and 40 km Thermonuclear Source

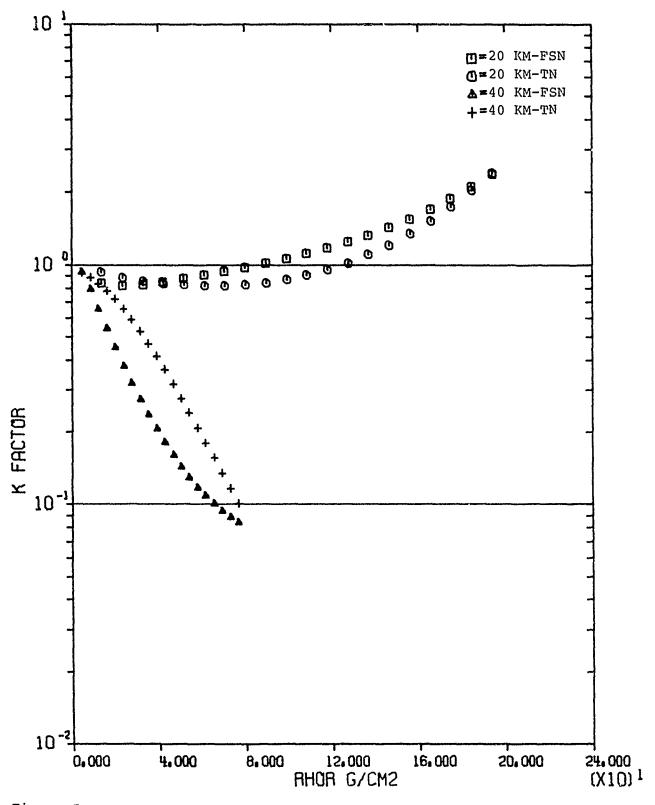


Figure 7. Coaltitude Neutron Silicon Dose K-factors at 20 and 40 km from both a Fission and Thermonuclear Source

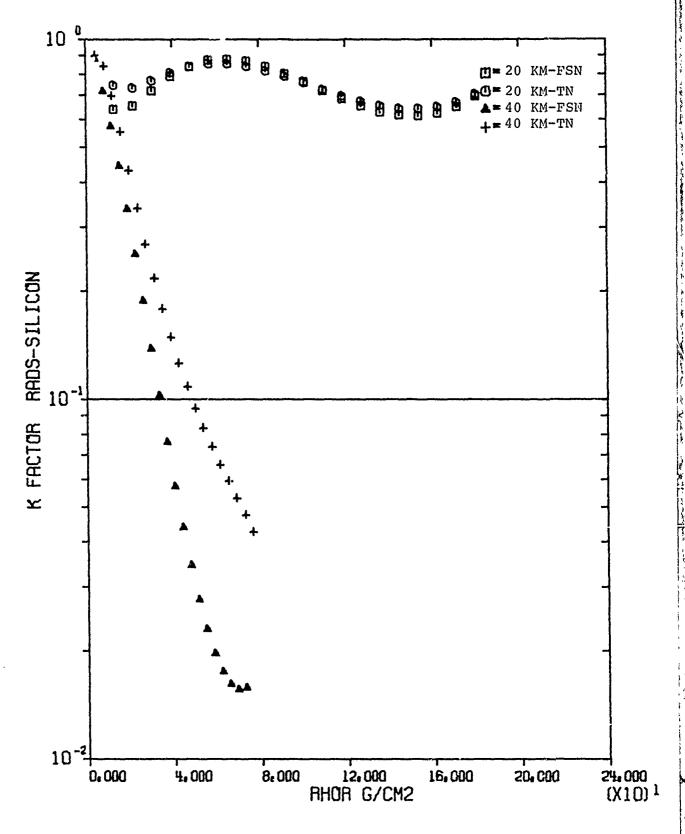


Figure 8. Coaltitude Secondary Gamma Silicon Dose K-factors at 20 and 40 km from both a Fission and Thermonuclear Source

4. SECONDARY GAMMA K-FACTORS AT SMALL MASS RANGES

There is one feature of the results for secondary gamma doses at the highest altitudes, 60 kilometers and above, that deserves an additional comment. Figure C-84 in Appendix C is a good example of this data for the case of a thermonuclear source at 80 kilometers. The silicon K-factors for the higher sampling altitudes are very large at low mass ranges. The K-factor, as was pointed out in section 3, is the ratio of the $4\pi R^2$ dose in real air to the $4\pi R^2$ dose in homogeneous air. The homogeneous results for this study were based on ANISN code calculations. Because of the uncertainties in these results at small mass ranges, most scaling codes use an interpolation method between 0.0 and about 1.0 gm/cm2. Such a method was used to produce the homogeneous data shown in Appendix A. No such approximations or assumptions were necessary in the MORSAIR results. should be noted that while the K-factors at these small mass ranges may not necessarily reflect differences between the homogeneous and real air results, they do reflect differences between the real air results and those from typical scaling codes.

SECTION V SUMMARY AND CONCLUSIONS

In this report we have presented the results of 15 MORSAIR Monte Carlo code calculations of neutron and secondary gamma transport in a real variable density atmosphere. two source spectra used in these calculations were an unclassified fission spectrum and an unclassified thermonuclear spectrum. The source altitudes varied from 5 to 80 kilometers and the energy dependent particle fluences were scored at more than 150 detectors located about the source for each run. spectrum at each detector was weighted with fluence, silicon, and tissue dose response functions. The $4\pi R^2$ dose data has been fit to a seven parameter equation in areal density at each sampling altitude of the MORSAIR run. Coefficients of these fits and plots of the silicon dose and K-factors have been presented. We have shown how the K-factor, the correction to the scaled homogeneous transport data to account for variable density atmosphere effects, depends on several parameters.

As stated at the outset, the objective of this study was to provide data which would lend some insight into the accuracy of mass integral scaling of homogeneous air transport data at high altitudes. The data presented in this report demonstrate that mass integral scaling can lead to large errors particularly for source altitudes greater than 20 kilometers. We believe these data can serve as the basis of an algorithm for predicting the correction to the mass integral scaled dose for sources and response functions similar to those used in this study.

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 Evaluation of the Spatial and Time-Dependent Neutron
 Environments in the Atmosphere from an Arbitrary
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 February, 1969.

APPENDIX A

ANISN HOMOGENEOUS AIR DATA

This appendix contains the coefficients of the fits to the neutron and secondary gamma transport ANISN code results provided by AFWL (ref. 12). The data was fit to the equation

ln $(4\pi R^2 \text{ Dose})$ = A + Bx + Cx² + Dx^{3/2} + Ex^{1/2} + Fx^{1/3} + G ln x where x is the mass range in gm/cm².

The coefficients of this fit for the different doses and sources are shown in Table 6.

The $4\pi R^2$ doses and fluences as calculated using these fits for both neutrons and secondary gammas have been plotted for the two source spectra in figures A-1 through A-6.

TABLE 6

ANISN HOMOGENEOUS AIR DATA

3746E+00	6
-,84022E-02	93271E+00
-,13975E+01	95495E+00
-,21249E+00	110916E+01
-,17644E+01	11093E+01
32101E+01	F.113945+01
15135E+01	575995+00
-16135E+02	13115+01
41090E+01	350675+01
37018E+01	339675+01
E.179245.01	E . 95659E+00
-131595.01	.65711E+00
-392145.01	.1336E+01
-270245.01	.25366E+01
-334265.01	.25306E+01
0	0
.15771E-02	.23939E-02
.1826E-02	.21001E-02
.39861E-02	.43226E-02
.4466E-02	.47309E-02
C 17913E-04	C27961E-04
22342E-04	24566E-04
54364E-05	37267E-04
54917E-04	55756E-04
53973E-04	55243E-04
B 97296E-01 98348E-01 16126E-00 17636E-00	B 90163E-01 79950E-01 11511E+00 16462E+00 16496E+00
A20795E-02 19711E-02 21750E-01 18463E-02 79627E-00	A 2528iE+02 25566E+02 48600E+01 26416E+02 26317E+02 57438E+91
SOURCE THERMONUCLEAR THERMONUCLEAR THERMONUCLEAR FISSION FISSION	CAPMAS SOURCE SOURCE THERMONICLEAR THERMONICLEAR FISSION FISSION
NEUTRONS DOSE SILICON TISSUE FLUENCE SILICON TISSUE FLUENCE	SECONDARY GAMMAS DOSE SOUR SILICON THERMON TISSUE THERMON SILICON FISSION FISSUE FISSION FLUENCE FISSION

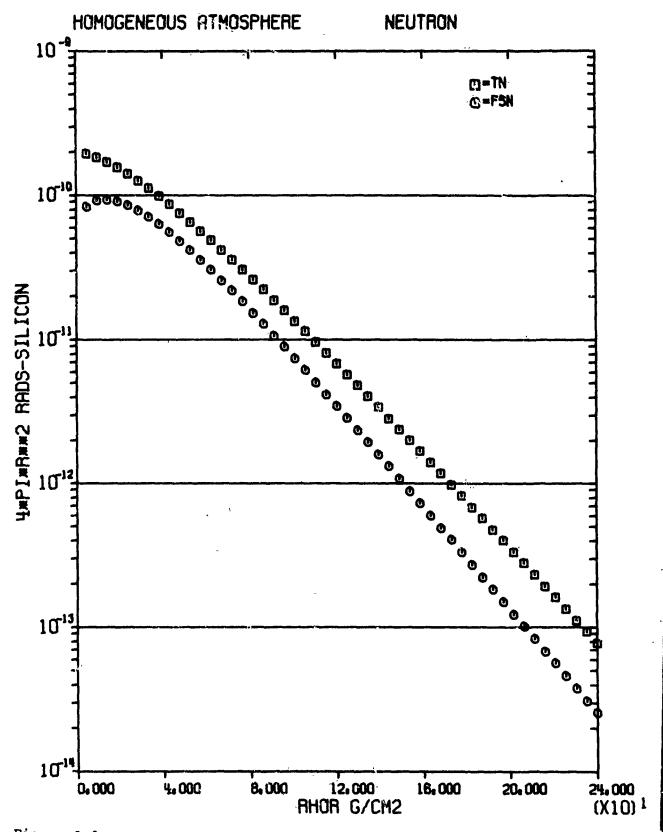


Figure A-1. Homogeneous ANISN Fit Data. $4\pi R^2$ Neutron Silicon Dose for a Fission and Thermonuclear Source.

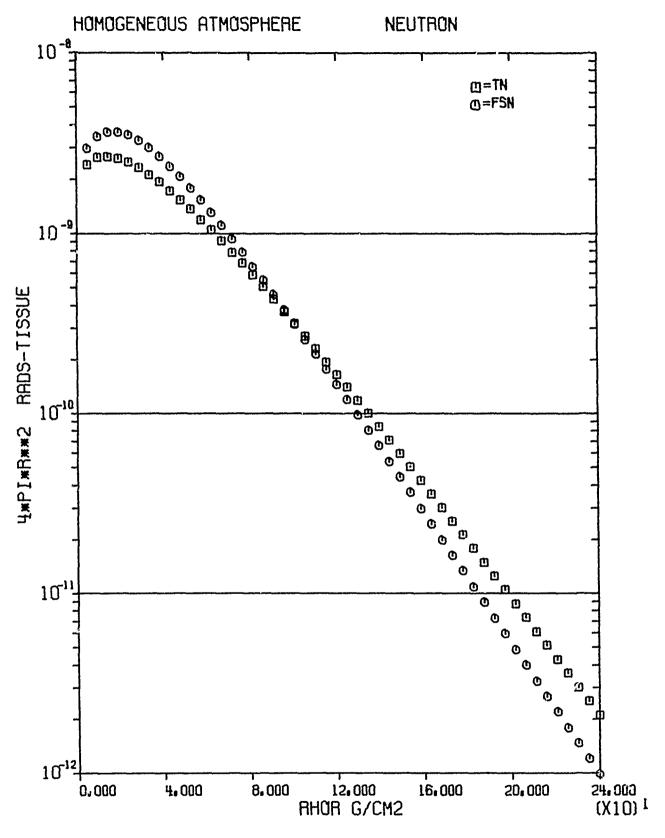


Figure A-2. Homogeneous ANISN Fit Data. $4\pi R^2$ Neutron Tissue Dose for a Fission and Thermonuclear Source.

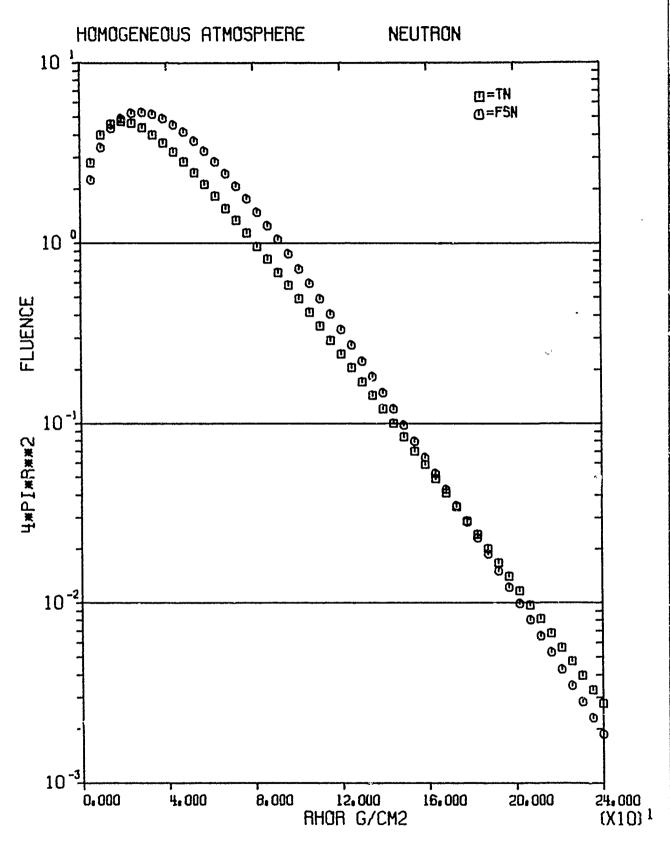


Figure A-3. Homogeneous ANISN Fit Data. $4\pi R^2$ Neutron Fluence for a Fission and Thermonuclear Source.

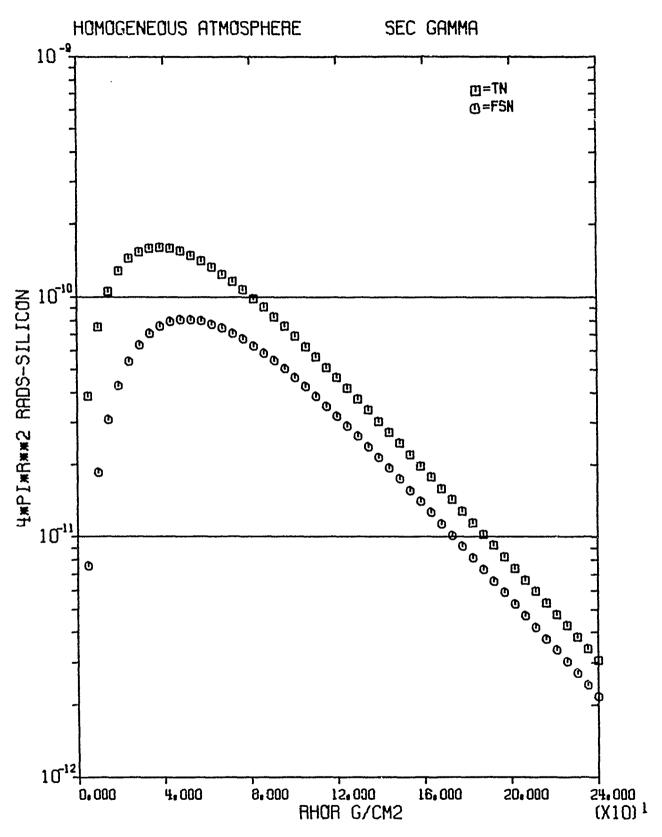


Figure A-4. Homogeneous ANISN Fit Data. $4\pi R^2$ Secondary Gamma Silicon Dose for a Fission and Thermonuclear Source.

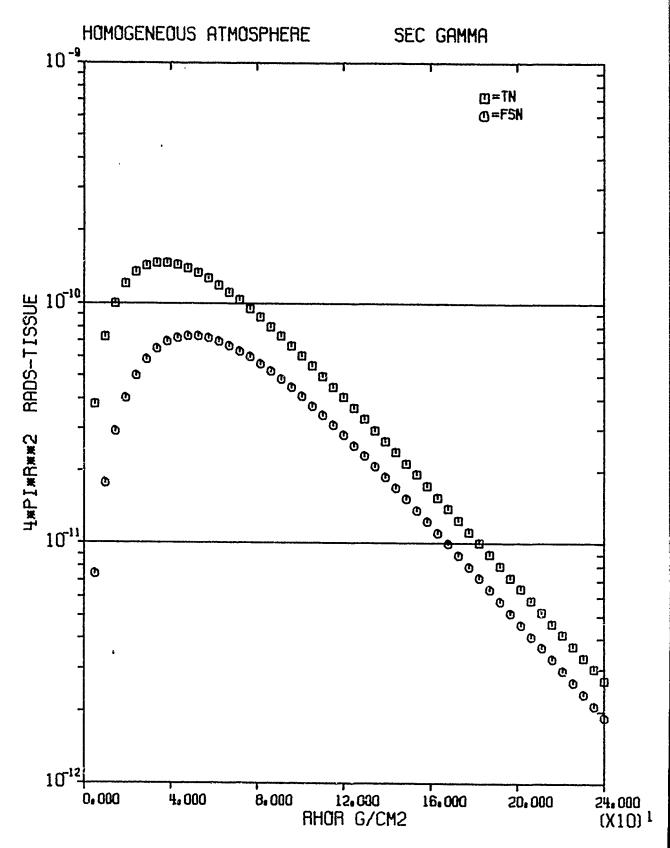


Figure A-5. Homogeneous ANISN Fit Data. $4\pi R^2$ Secondary Gamma Tissue Dose for a Fission and Thermonuclear Source.

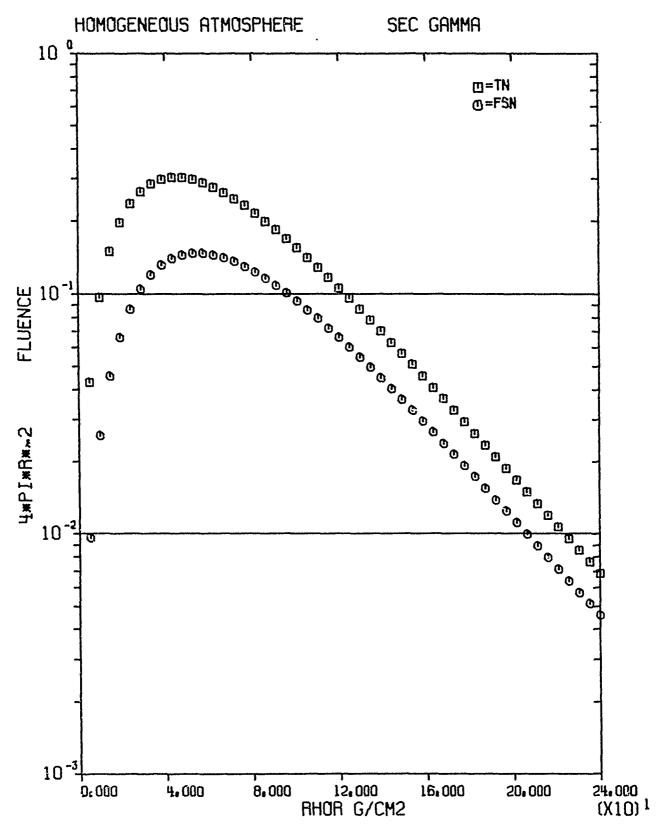


Figure A-6. Homogeneous ANISN Fit Data. $4\pi R^2$ Secondary Gamma Fluence for a Fission and Thermonuclear Source.

APPENDIX B FITS OF MORSAIR REAL AIR DATA

This appendix contains the results of a weighted least squares fit to the MORSAIR data. For each of the 15 MORSAIR runs, the Following data is given at each sampling altitude:

- 1. The sampling altitude in kilometers.
- 2. The dose response function.
- The minimum and maximum mass ranges at which the fit is considered valid.
- 4. The coefficients to the fit equation:

In
$$(4\pi R^2 \text{ Dose})$$
 = A + Bx + Cx² + Dx^{3/2} + Ex^{1/2} + Fx^{1/3} + G ln x where x is the mass range in gm/cm²

5. The root mean square percentage difference between the fit values and the actual MORSAIR data.

These data are shown for both the neutron and secondary gammas.

TABLE 7

FIT COEFFICIENTS FOR MORSAIR RUN NO. 1 FISSION SOURCE IN A HOMOGENEOUS ATMOSPHERE

C	-41939E-01 -340524E-01 -39339E-01 -38146E-01 -59657E-01 -39657E-01 -39657E-01 -39103E-01 -43433E-01	.21409E+02 .37856E+02 .35968E+01 .22411E+02 .19000E+02 .23513E+02 .23513E+02 .20111E+02 .2182E+02 .17816E+02	-21409E+02 -17826E+02 -3596E+02 -2241BE+01 -13396E+01 -23513E+02 -23513E+02 -22879E+00 -22879E+00 -17816E+02
FIT COEFFICIENTS C D E		æ	83
		21348E-01	21
0. 0. 0.		-,21345E-01 -,21479E-01	345E-01 479E-01
19673E-01		.14821E+00	174
19594E-01		.14653E+00	
	•	0145598400	
27023E-01	• •		.46736F+00
24526E-01	•	•	.43522E+00
•	ċ	22049E-01 0.	0496-01
•0	ċ		22352E-01
•0	á	21530E-01 0	530E-01

TABLE 8

FIT COEFFICIENTS FOR MORSAIR RUN NO. 2 FISSION SOURCE IN REAL AIR AT 5.0 km

RHS PCT DIFF	0 00. 88 45 88 00. 88 45 88 05 46 88 05 88	RMS PGT 01FF 4482 4494 314 3207 413 413 320	
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FIT COEFFICIENTS 0	0. 0. 0. 69318E-03 43986E-03 43986E-01 0. 12405E-01 0. 0. 0. 0.	FIT COEFFICIENTS 0.0000 0.0000 0.0000 0.0000 0.0000 0.00000 0.000000	•
FIT	0. 19830E-04 15472E-03 19830E-03 19334E-03 62790E-03 62790E-03 22751E-04 12823E-04		23932E-04
ω	-,431676-01 -,417386-01 -,439306-01 -,326886-01 -,444656-02 -,713576-01 -,47716-01 -,606386-01 -,444846-01 -,419196-01	1.2.2.0 1.2.2.0 1.2.2.0 1.2.0.0 1.2.0.0 1.3.0.0 1.3.0.0	12186E-01
⋖	212146+02 176416+02 116416+01 219336+02 186226+02 151826+01 233166+02 197526+02 17596+02	A21853E+0231667E+0024685E+0224764E+0231933E+0125891E+0125891E+0122037E+0222027E+0222027E+0222027E+0222027E+02	88356E+00
HAX KHOR GH/CH2	193.800 193.800 193.800 193.900 193.900 193.900 194.100 194.100 194.100 197.800	HAX RHOR GH/GM2 193.600 193.800 193.800 193.900 193.900 194.100 194.100 197.800	197.800
HIN RHOR GH/GH2	5500 5000 6000 6637 6637 6637 6637 6637	HHOR CH2 5500 5500 0000 0000 0000 0000 0000 00	06.020
NEUTRONS SAHPLING ALTITUDE (KH)	3.68 SILICON 3.68 TISSUE 3.68 TISSUE 4.40 SILICON 4.40 FLUENCE 5.07 SILICON 5.07 TISSUE 5.91 TISSUE 5.91 TISSUE 5.91 TISSUE 5.91 TISSUE 5.91 TISSUE	SECONDARY GAHHAS SAHLING ALTITUDE (KM) 3.68 SILICON 106- 3.68 TISSUE 106- 3.68 FLUENCE 106- 4.40 SILICON 47- 4.40 SILICON 6- 5.07 SILICON 6-	.91

TABLE 9

FIT COEFFICIENTS FOR MORSAIR RUN NO. 3 FISSION SOURCE IN REAL AIR AT 10 km

RMS PCT DIFF	. 7002 . 7006 . 5006 . 3494 . 343 . 343	RHS PCI 194 .234 .362 .349 .558 .558
u		o - • • • • • • • • • • • • • • • • • • •
L.		
w		ш
FIT COEFFICIENTS D	0. 0. 0. 0. 0. 0. 0. 0. 0. 0. 0. 0. 0. 0	FIT COEFFICIENTS 4 0. 0. 0. 0. 0. 0. 0. 0. 0. 0. 0. 0. 0.
FIT 0	.43441E-04 .57461E-04 .40175E-04 .32852E-04 .20277E-04 .24591E-03 .27101E-03 .64582E-03 .64582E-03	FIT 16085E-04 17352E-04 11927E-04 51383E-04 5137E-04 51386-04 10726E-02 10650E-02 10479E-02 14136E-05 4436E-05 48572E-05
c	533726-01 561276-01 506216-01 453306-01 426406-01 376916-01 .127166+00 293016-01 345516-01	B -16039E-01 -17017E-01 -78809E-02 -84583E-02 -31296E+00 -22918E+00 -22918E+00 -22918E+00 -22918E+00 -22918E+00 -22918E+00
⋖	20591E+02 16490E+02 -44926E+01 21491E+02 17849E+02 3325E+02 1886E+02 23255E+02 18146E+02 22233E+02 22233E+02	A 22265E+02 22397E+02 22683E+02 22768E+02 22768E+02 24109E+01 26343E+02 26343E+02 26349E+01 21546E+02 30801E+00
HAX KHOR GH/GH2	194.200 194.200 194.200 189.200 197.200 194.700 194.700 194.700 194.700 196.200	MAX RHOR GM/CM2 194.200 197.200 197.200 197.200 197.200 194.700 194.700 194.200
HIN RHOR GH/CH2	210 210 210 2210 2250 2250 2288 376 1990	HHINN CHORN CHORN CHO CHO CHO CHO CHO CHO CHO CHO CHO CHO
NEUTRONS SAHPLING ALIITUBE (KM)	7.45 SILICON 7.45 FILICON 7.45 FLUENCE 8.84 SILICON 8.84 TISSUE 8.84 FLUENCE 10.00 FLUENCE 12.13 SILICON 12.13 FLUENCE 12.13 FLUENCE 12.13 FLUENCE	SECONDARY GAMMAS SAMPLING ALITIUDE (KM) 7.95 SILICON 98. 7.95 FLUENCE 98. 7.95 FLUENCE 98. 8.84 SILICON 53. 8.84 FLUENCE 57. 10.00 FLUENCE 65. 12.13 SILICON 78. 12.13 TISSUE 69.

TABLE 10

FIT COEFFICIENTS FOR MORSAIR RUN NO. 4 FISSION SOURCE IN REAL AIR AT 15 km

į	RHS PCT DIFF	. 973 . 779 . 400 . 387 . 412 . 314 . 512 . 520 . 520 . 738	RMS PCT DIFF .295 .354 .461 .684 .708 .225 .235 .532 .532 .345
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	ls.		0 d d d d d d d d d d d d d d d d d d d
	w		m
	COEFFICIENTS D	0.000.000.000.000.000.000.000.000.000.	COEFFICIENTS 0.0000 0.00000 0.00000 0.000000
	FIT	0.00 0.00	FIT 0.00000000000000000000000000000000000
	83	32504E-01 33576E-01 22854E-01 26164E-01 26164E-01 34443E-01 34443E-01 3443E-01 4612E-01 46112E-01 46112E-01 46378E-01 3692E-01 3692E-01	-21256E-01 -21397E-01 -17650E-01 -1827E-01 -1827E-01 -20949E+00 -20949E+00 -20949E+00 -22972E+00 -11860E+00 -27679E-01 -27679E-01 -27679E-01 -11868E+00 -11868E+00 -11868E+00 -11868E+00 -11868E+00 -11868E+00 -11868E+00 -11868E+00 -11868E+00
	⋖	22760E+02 35957E+01 2270E+02 18771E+02 3280E+02 1978E+02 1978E+02 19110E+02 19110E+02 19110E+02 22994E+02 25994E+02 25994E+02 25994E+02 25994E+02 25994E+02 25994E+02 25994E+02 25994E+02 25994E+02 25994E+02 25994E+02 25994E+02 25994E+02 25994E+02 25994E+02 25994E+02 25996E+	A A A S A S A S A S A S A S A S A S A S
	HAX RHOR GHZCM2	,	MAX RHOR CH/CN2 195.600 195.600 194.900 194.900 198.000 198.700 195.700 195.700 195.700
	HIN RHOR GH/CH2	90000000000000000000000000000000000000	MINUR CH2
NEUTRONS	SAMPLING ALTITUDE (KH) DOSE	D SILICON D SILICON D TISSUE D FLUENCE D	SAMPLING ALITUDE (KM) DOSE 11.30 SILICON 11.30 SILICON 11.30 FLUENCE 12.80 SILICON 12.80 SILICON 12.80 FLUENCE 12.80 SILICON 15.29 FLUENCE 15.29 SILICON 15.29 FLUENCE 17.37 FLUENCE 17.37 FLUENCE 21.66 FLUENCE 21.66 FLUENCE 21.66 FLUENCE 21.66 FLUENCE 21.65 SILICON 21.66 FLUENCE 21.65 SILICON 21.65 FLUENCE 25.51 FLUENCE

TABLE 11

FIT COEFFICIENTS FOR MORSAIR RUN NO. 5 FISSION SOURCE IN REAL AIR AT 20 km

RMS PCI UIFF	2.151 2.288 .617 .491 .224 .364 .366	. 4116 . 347 . 347 RMS PCI	01FF • 388 • 376 • 358 • 561 • 560 • 402 • 255 • 267 • 207
ن		 	••••••••••••••
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ω		ш	
COEFFICIENTS 0	0. 0. 0. 0. 0. 0. 0. 0. 0. 0.	S	0. 0. 0. 0. 0. 0. 0. 0. 0. 0. 0. 0. 0. 0
FIT	0. 0. 174422-04 -13448E-04 -3666E-03 -29546E-03 -2234E-03 -24718E-03 -24718E-03 -26595E-03 -49685E-03 -4199E-03		6
3	45672E-01 44564E-01 31650E-01 29758E-01 35143E-01 35143E-01 31543E-01 .41472E-01 .11808E+00 .12286E-01 .17341E-01	1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1	29594E-01 27217E-01 130961E-01 15237E-01 22272E+00 .21781E+00 .23965E+00 .15749E+00 .15749E+00 .15749E-01 .51479E-01 .51479E-01 .51479E-01
⋖	22356+02 17028E+02 20633E+01 22626E+02 18610E+02 19776E+02 19776E+02 19776E+02 19776E+02 19776E+02 19776E+02 19776E+02 19776E+02	18344E+02 .2269E+01 21063E+02 17345E+02 .36959E+01	-21067E+02 -21659E+02 -22659E+01 -226597E+02 -11193E+01 -22643E+02 -11193E+01 -26436E+02 -26436E+02 -26436E+02 -26436E+01 -26436E+01 -26405E+01 -26076E+01 -26076E+01 -24076E+01 -24076E+01 -24086E+01 -23905E+01 -23002E+01
HAX RHOR GH/CH2	195.400 195.400 200.000 185.700 185.700 193.900 193.900 193.600 190.000 180.000	193 193 196 196 196	64/CN2 195.400 195.400 195.400 200.000 200.000 193.900 195.400 195.400 195.400 195.400 195.400 195.400 195.400
MIN RHOR GM/GH2	146.500 146.500 17.770 77.770 77.770 77.770 77.770 10.690 13.370 13.370	1110 9110 910 910 910 910	6H/CH2 146.500 146.500 146.500 77.77 77.770 77.770 77.770 77.770 4.149 10.690 13.370 13.370 13.370 13.370 13.370 13.370 13.370 13.370 13.370 13.370 13.370 13.370
NEJTRONS SAMPLING ALTITUDE (KM) DOSE	SILICON TISSUE FLUENCE SILICON TISSUE FLUENCE SILICON TISSUE FLUENCE SILICON TISSUE FLUENCE SILICON	26-13 26-13 34-72 34-72 34-72 34-72 SECC	ALITIUDE (KH) DOSE 12.10 SILICON 12.10 TISSUE 14.85 SILICON 14.85 FLUENCE 14.85 FLUENCE 20.61 TISSUE 20.61 TISSUE 22.08 FLUENCE 23.77 FLUENCE 24.72 FLUENCE 34.72 FLUENCE

TABLE 12

FIT COEFFICIENTS FOR MORSAIR RUN NO. 6 FISSION SOURCE IN REAL AIR AT 30 km

RMS PCI OIFF	1.015 1.036 1.036 .306 .253 .284 .284 .221	RHS PCT NIFF	.317 .341 .341 .357 .357 .357 .533 .524 .524 .321
s		u	
u.		u,	0. 0. 0. 0. 0. 0. 1.236856+01 1.236836+01 0. 0. 0. 0.
พ	0. 0. 0. 0. 0. 33558E+00 0. 14633E+01 0. 0. 0.	W	0. 0. 0. 33043E+01 .35687E+01 .36873E+01 0. 0. 0.
FIT COEFFICIENTS 0	0. 3.3264E-02 .64447E-02 .64447E-02 .86540E-02 .13892E-01 .17316E-01 -56114E-02 -611016E-02 -10205E-01 .47552E-02 .88411E-03 .47552E-03	r coefficients O	0. 0. 10. 10. 10. 10. 10. 10. 10
c FII	0. 1.94535E-04 194575E-03 11746E-03 41796E-03 41796E-03 41796E-03 41590E-03 1190E-03 11662E-03 11662E-03 11662E-03 11662E-03	FIT C	0. 151010E-03 15182E-03 26976E-03 10419E-03 77261E-04 -24801E-03 51697E-03 51697E-03 51897E-03 45898-03 45898E-03 41604E-03 -21958E-03
œ	397146-01 375866-01 312696-01 584536-01 685746-01 182636+00 273686+00 273686+00 273686-01 174746-01 174746-01 174746-01 174746-01 174746-01 174746-01 191396-01 35396-01 35396-01	ω	23418E-01 24261E-01 18617E-01 .12850E-01 .10470E-01 31042E+00 31832E+00 402832E+00 40283E-01 .1145E+00 .80970E-01 .1145E+00 .80970E-01 .1145E+00 .80970E-01
વ	21952±402 235835+02 222556+02 180676+02 242736+02 242736+02 210165+02 210165+02 210165+02 228056+02 228056+02 228056+02 24406+02 24606+02 222206+02 222206+02 222206+02 222206+02	વ	22240E+02 1590E+02 1590E+01 2343E+02 23413E+02 23413E+02 26257E+01 26350E+02 28390E+02 24982E+02 24982E+02 24982E+02 24982E+02 24982E+02 24982E+02 24128E+02 24128E+02
MAX RHOR GH/CH2	192-500 192-500 192-500 180-000 180-000 180-000 200-000 200-000 170-400 170-400 80-000 80-000 81-000 119-000	HAX RHOR	
MIN RHOR 6H/CH2	1114-200 1114-200 1114-200 46-240 46-240 46-240 46-240 1-755	HHAS HIN RHOR	71 71 71
NEUTRONS SAMPLING ALTITUDE (KM) DOSE	15.00 SILICON 15.00 FLUENCE 20.00 FLUENCE 20.00 TISSUE 20.00 FLUENCE 30.00 FLUENCE 30.00 FLUENCE 40.00 FLUENCE 40.00 FLUENCE 50.00 FLUENCE 50.00 FLUENCE 50.00 FLUENCE 50.00 FLUENCE 60.00 SILICON 60.00 FLUENCE 50.00 SILICON 60.00 FLUENCE 50.00 FLUENCE 60.00 FLUENCE 60.00 FLUENCE	SECONDARY GAHHAS SAHPLING ALTITUDE R	5.00 SIL 5.00 SIL 0.00 IIS 0.00 IIS 0.00 FLU 0.00 SIL 0.00 SIL 0.00 SIL 0.00 SIL 0.00 SIL 0.00 SIL 0.00 SIL 0.00 SIL

TABLE 13

FIT COEFFICIENTS FOR MORSAIR RUN NO. 7 FISSION SOURCE IN REAL AIR AT 40 km

RHS PCT DIFF	4.12	. 242 . 162 . 243	.141 .205 .080	.148	RMS PCT DIFF	.284	.338 .331 .363 .364	. 222
و			*	• • • • • • • • • • • • • • • • • • • •	y	• • • • • • • • • • • • • • • • • • • •		
U.	• • •				u,	20000	.15262E+02 .15395E+02 .16587E+01	
ы		0. 0. .56989E+00. .81747E+00	.12907E+0		w		96707E+01 98482E+01 .18255E+01 0.	
FIT COEFFICIENTS 0	0.00	14985E-01 11245E-01 11643E-01 .28186E-02	.17118E-01 34911E-01 42967E-01 35842E-01	32016E-01 21719E-01 42014E-01 51544E-01	FIT COEFFICIENT O	0. 0. 0. -37945E-01 -38040E-01	14347E+00 14660E+00 -12357E+00 -12357E+00	14674E+00 14674E+00 14902E+00 13592E+00 13911E+00
: :	.82744E-04 .75636E-04 .37861E-04	.88112E-03 .69360E-03 .64291E-03 .19530E-03	83743E-04 83743E-04 .36063E-02 29530E-02	.33967E-02 .30257E-02 .42906E-02 .57285E-02	S.	.21866E-04 .25265E-04 .27114E-04 .17652E-02	.74233E-02 .742426E-02 .745426E-02 .14518E-02 .93974E-02	.11449c-01 .13739c-01 .13961c-01 .15816c-01 .14306c-01
83	5868E-01 2914E-01 1547E-01	7457E-01 3676E-02 3320E-01 4843E+80		312425-02 500606-01 373956-01 411906-01 207206-01	ത	25431E-01 26393E-01 24513E-01 .19379E+00	10333E+0 10333E+0 10604E+0 37592E+0 39051E+0	.42695E+UU .36778E+UU .37548E+UU .39940E+UU .28517E+UU .29520E+UU
₹	000	23292E+02 19568E+02 .16180E+01 23855E+02	9999	-119372E+02 -11083E+01 -22998E+02 -19391E+02	ধ	22869E+02 22953E+02 14491E+01 25957E+02	-,4568uc+41 -,3389246+02 -,33895E+02 -,971,92E+01 -,25987E+02 -,26113E+02	485/5E+01 25552E+02 25594E+02 44712E+01 25204E+02 25234E+02
HAX RHOR GH/CN2	179.300 179.300 179.300	120.600 120.600 120.600 76.530	76.530 42.630 42.630 42.630	32.420 32.420 32.420 34.550 24.550 24.550	NAX RHOR GH/CM2	9.300 9.300 9.400 0.600	2.660 2.660 2.660 2.630 2.630	530 420 420 550 550 550
HIN RHOR GM/CH2	54.830 54.830 54.830	10.110 10.110 11.610 .750	**************************************	3.661 3.654 3.674 3.674 3.674	MIN RHOR GH/CM2			4.985 3.661 3.661 3.661 3.674 3.674
NEUTRONS SAMPLING ALTITUDE (KM) DOSE	000	20000	20000	• • • • • •	SAMPLING ALTITUDE (KM) DOSE	000000	30.00 FLUENCE 40.00 SILICON 40.00 TISSUE 60.00 SILICON 60.00 SILICON	

TABLE 14

FIT COEFFICIENTS FOR MORSAIR RUN NO. 8 FISSION SOURCE IN REAL AIR AT 60 km

RHS	PC1 01FF	.658	i !	.084	.115		. 079	.110		.034	S		• 022	• 033	,	. 030	• 036			RMS	PCT	DIFF	.450	• 455		. 260	.196		901.	.100	500	•		-	9 0	007.	9	07.			
ŧ	ပ	• •							•		•	.0	.0	•	•	•	٠.	•0			و			0.	•	•	•	•	•	•	•	•	•	•	•	•	•	•	•	.0	
ı	l L	•	•							0,0	•	•	•	•	•	•	•	•			u.		•		•	•	•	•								. 31852E + U 3		141/8E+US			
	រោ	• 0					00+355486	24646E+0P	.50382E+00	.10101E+01	.23267E+01	.52480E+01	,31606E+01	.37548E+01	.63111E+01	.11842E+01	.33351E+01	.77475E+01			ω		.0	0.		0.	0.	•								33943E+US		36E+03	25895+03	33oE+03	
COEF	a		•	5	- 07582F-01	14184F + 01	00-340444	355255+00	54993F+00	-471655+00	.91824E-01	.12877E+01	.11588E+01	.11755E+01	.25501E+01	-,61196E+00	.12769E+01	.49148E+01		COEFFICIENTS			0.	0.	0.	01E+0	91E+0	32784E+00	84E+01		41E+01					31936E+G2		9346E+0	955E+0	1301E+0	
FII	Ų	20671E	20-346622		20-126-000	13710F=0t	10 101 101	. 58082F-01	- 940 645-03	.13000E+00	.43052E-01	12917E+00	16316E+03	.12761E+00	.344 80E+00	.25821E+00	15875E+00	0953E+00		FII	ပ		+0-	+0-	70-	-01	-01	-01	+00	.35979E+00	00+	+00	D :	00+	+01	.45831E+01	99816+01	•		-,59583E+01	
	മ	351E-0	665223E=01	10-302000+	100325700	305/125400		. 4.232.25±00	58734F+60	. 56 8 8 0 F - 0.3	13417E+01	43113E+01	30714E+01	35068E+01	65347E+01	28099E+00	34546E+01	97723E+01			æ		32548E-01	36515E-0x	6	. 97 95 BE+00	.97265E+00	.90362E+00	15385E+02	14848E+N2	16180E+02	•11359E+02	.11582E+02	.23989E+01	.93093E+02	.10200E+03	.65867E+UZ	10869E+03	11853E+03	b6491E+02	
	Ф	21878E+02	→ :	0.000	70.4076407				. 4	۰ د	+92	+07	4269E+02	0960E+02	4938E+01	3662E+02	.20682E+0	.15379E+0			A		22825£+02	-,22795E+02	18118E:01											79323E+02					
MAX	RHOR GN/CN2	108-100	108-100	000 .00T	166.00	26.990	066.07	9.715	77.0	5,139	5,139	5-139	3.451	3.451	3.451	2.484	2.484	\$		×	KHOK	GH/CH2	108.100	08-100	8.100	966 • 9	9.990		9.715							3.451			2.484	484	•
MIN	RHOR GM/CH2	56.790	062.9	0.00				2000	2 4	0 CT •	34.8	318	286	286	286	287	287	.287	HAS	7	RHOR	GH/CH2		56.798			889	_	150	.150	.150	.318	.318	.318	.286	.286	•286	-287	.287	.287	ļ
NEUTRONS SAMPLING	ALTITUDE (KM) 00SE	IS	0000	3	•	3 6	3	•	3 6			00.00		100.00				000	SECONDARY GANMAS	200	ALTITUDE	(KH) DOSE	NO STATEON	ON TISSUE	D. DO C. LENCH	0.00 SILICON	0.00	00.0	00.0	60.00 TISSUE	00.0	0.00	0.00	0.00	0.00	0.00	80.09	0.02 SI	2	0.03 FL	

TABLE 15

FIT COEFFICIENTS FOR MORSAIR RUN NO. 9 FISSION SOURCE IN REAL AIR AT 80 km

,	RMS PCI OIFF	.674	144	. 210		.052			• 100	. 032	240.	•	• 115		.013	02			RHS	PCT	OIFF	. 641	.660	C	467.	+62.	.173	.170		.125	.121	100	460.	•	• 054	640.			,	
	3			• •		• •	.0	•	• 🗖	• •			•			•				ى		0.	0.	• 0	•	•	•	•	c.	•	• •	•	• •	• •	0.	0.	•	•	•	•
	u.	• • •	•	• •	0.	• •	0.	•	•	• •	0.	•0	• 0	•	•	• •	• 0			L.		0.	• 5			•	28225E+0	9	16182E+0	65661E+0	9	39544E+0	4756.05 +0	2 2	58885E+0	54448E+0	5	.12190E+64	2 9	2
	ш	ŋ. 0.	•		0.	.23818E+01	.45133E+01	32355E+00	43704E+00	/U//5E+UG	.55429E+01	.98539E+51	.15278E+02	.1/11/E+U2	2023/2102	. 10591F+02	.17012E+02			w		•0		•	•	•	. 74.798£+03			63316E+02	58011E+02	18497E+02	* < 75555 F + 82	- 444059E+U4	10062E+04	92326±+03	45605E+U3	22419E+04	#0+300#TZ*-	-,135435704
	FIT COEFFICIENTS D	•••	0.	20815E+00															COFFETCIFNIS	0					0.0	000) (20	02	02	02	200	2 5	9 6	40	70	03	58120E+04	7 .	‡
	C FII	78435F-02 46094E-02					14925E+00						.13653E+03		.92046E+02				113		ı						.8642/E-UI		.48666E+01				•11406E+03					.44316E+04		.234522+04
	œ	.91675E+00 .52232E+00	585E+0	.62712E+00 .38252E+00	607E+0	535+0	844F+0	834E+0	90E+0	502E+0	720E+0	532E+0	950E+0	30E+0	281E¢0	8419540	33E+0 52E+0			œ	ı	92E+0	.21340E+0	138E+0	532E+0	624E+0	9E+0	30 AE + 0	447E+0	428E+0	5E+0	343E+0	+28E+0	319E+U	13184F+0	.11891E+04	301E+0	.35210E+04	3746E+U	.19417E+U4
	⋖	51858E+02 36148E+02	LE+02	23769E+02 19987E+02	¥ + 0	73		23515E+02	20070E+02	53213E-01	20525E+02	E+0	+25E+0	Z +0	+524E+0	050E+0	1035E+0			٥	ſ	.34551E+02	.40026E+02	.15987E+02	26538E+02	26578E+02	58976E+U1	783555401	.10065E+02	38447E+02	38282E+02	15798E +02	-,35340E+02	34753E+02	642845402	62926E+02	29660E+02	95476E+02	92801E+02	51749E+02
	NAX RHOR GH/CH2	64.860 64.860	. 860	006	510	707	101	832							.153	.114	114		2	404	GH/CH2	660	9	860	900	006	006	# 4 5 C	† †	832								.114		
	HIN RHOR GH/CH2	56.90J 56.900	'u	3.008	3.087	267	197.	650.	• 059	•100		015	.015	.015	.015	.015	.015	HAS	2	OTE O	6H7CH2	ď	56.900	9	m	3.008	3.008	192.	.267	.020	•020	•020	•015	.015	0.40	.015	• 015	•015	.015	.015
NEUTRONS	SAMPLING ALTITUDE (KM) DOSE	20.00 SILICON	0.00	40.00 SILICON		0.0	00.0		0.00	80.00			0	00	00	140.00 SI	00.00	SECONDARY		SAMPLING	DOSE	MOSTITCON) H	00.00	0.00	.00	0.00	9	60.00 FLUENCE	00	00.	.00	00.00	00.00	00.00		20.00	40.00	0.00 TISSUE	.00

TABLE 16

FIT COEFFICIENTS FOR MORSAIR RUN NO. 10 THERMONUCLEAR SOURCE IN A HOMOGENEOUS ATMOSPHERE

RHS PCT DIFF	. 831 . 310 . 310 . 272 . 208 . 228 . 639	RNS PCT DIFF .395 .369 .328 .328 .402 .402
ు	••••••	o - • • • • • • • • • • • • • • • • • • •
ti.		F 0.000 0.000 0.22353E+02 0.22351E+02 0.22351E+02 0.000
w	0. 0. 0. 0. 531146-01 53194E+00 .90695E+00	E 0. 0. 0. 0. 0. 12623E+02 12618E+02 11079E+02 0.
FIT COEFFICIENTS 0	0. 0. -47837E-02 -64761E-02 -96085E-02 -7114E-02 -1508E-02 -15568E-02 -16031E-02 0.	FIT COEFFICIENTS 0. 0. 0. 0. 311961E-01 312070E-01 337779E-01 337779E-01 337279E-01 0. 0.
FIIT 0	10. 15232E+03. 19553E+03. 33873E-03. 10865E-03. 25930E-03. 29659E-04.	FIT C 0. 39443E-03 47454E-03 91430E-03 91430E-03 .89528E-03 .76242E-03
ω	-40110E-01 -37306E-01 -33455E-01 -43977E-02 -23693E-01 -364645E-01 -11062E-01 -401679E-01	B 23424E-01 22957E-01 24603E-01 .7339E-01 .10796E+00 .64638E+00 .64134E+00 .53115E+00
∢	20836E+02 18043E+02 .26092E+01 22309E+02 19739E+02 15816E+01 22338E+02 20338E+02 40209E+02 4020	A 20982E+02 21172E+02 .67622E+00 2336E+02 2336E+02 25099E+01 36942E+02 36942E+02 36942E+02 36942E+02 25099E+01
HAX RHOR GH/CH2	192.600 192.600 192.600 192.900 192.900 177.500 200.000 200.000 200.000	MAX RHOR GN/CM2 192-600 192-600 192-900 192-900 192-900 200-000 200-000 200-000 200-000
MIN RHOR GH/CM2	101.600 101.000 21.800 21.800 21.800 21.800 3.647 3.647 8.737 101.600	CHH CHH CHON CHOO CHOO COO CCOO CCOO CCO
NEUTRONS SAMPLING ALIITUDE (KM) DOSE	3.11 SILICON 3.11 TISSUE 3.11 FLUENCE 3.83 SILICON 3.83 FLUENCE 4.01 SILICON 4.01 TISSUE 4.92 SILICON 4.92 SILICON 4.92 SILICON 4.92 FLUENCE 4.92 FLUENCE 4.92 FLUENCE	SAMPLING ALTITUDE (KM) DOSE GM/ (KM) DOSE GM/ 3.11 SILICON 101. 3.11 TISSUE 101. 3.11 FLUENCE 101. 3.83 TISSUE 21. 3.83 TISSUE 21. 4.01 TISSUE 21. 4.01 TISSUE 21. 4.01 TISSUE 21. 4.02 SILICON 6. 4.92 FLUENCE 101.

TABLE 17

FIT COEFFICIENTS FOR MORSAIR RUN NO. 11 THERMONUCLEAR SOURCE IN REAL AIR AT 20 km

RHS	DIFF	1.548	.729	. 602		.224	. 265		. 393	.273		.507	. 302		. 415	. 395			RMS	PCT	DIFF	. 425	. 535	. 382	. 423		124.	565		.376	. 372		. 291	.294		.275	•275	
3	•	• •		0.	.0	•	0.	0.	.0	.0	•		g.	•		0.				9		•	.0	.0	•	0	0.	ن ن	0.	•	0.	• 0	0.	•	.0	.0	•	•
u	•																			u.																		
		000	.		0	0	•	• •	0	.	0	• •	0.	0.	0	0	0					0	9		•0	•	0		0	• 0	0	• 9	°0	.	0			
u.	ı			•	•	•		•	•ر		•	•	•	•	•		•			W		•	•	•	•		•			•		•		•	•		•	•
CLENTS		000	. 0	0	0	E-02 0	E-02 0	E-01 0	E-02 B	E-02 .0	E-02 0	E-01 0	E-02 0	E-02 0	E-02 D	E-02 0	E-02 0		COEFFICIENTS			0	0	0	0		10	15	.01	11	01	01	10	01	0,1	20.	20	95
FIT COEFFICIENTS	•	.0	0.	ن ن	•	49696	-,89599	16821	6940	29363	75428	.12321	.55899	26444	.13831	59669*	.34209			0			0.	•0	•	0.	20479E	19732	25783	14523	14192	17928	11795E-	10955	11635	74619	79559	45256
I d			997E-05	57039E-05	334E-05	781E-03	374E-03	211E-03	30 E-0	32E-0	36E-0	31E-0	J6E-0	52E-0	189E-04	51E-0	30E-0		FIT	ပ				-04	-04	- 05	-03	-03	-03	-03	.56E-03	162E-03	100E-03	105E-03	1284E-03	946	165	946
		•		i	i	٠٠٠	•	ų	,,	•	•	1.1	;	1	•	- 1	1.1					0	•	.222	.203	•							.42800E			٠,	σ	.164
α	3	42995E-01		34334E-01			.35031E-01	.88305E-01	70137E-01	50342E-02	.21521E-01	12496E+00	76521E-01	71574E-01	60655E-01	99338E-01	73619E-01			හ		15299E-01	8490E-3	6594E-0	26885E-01	19313E-01	•14062E+00	.13469E+00	.18342E+00	.98639E-01	• 95559E-01	.12772E+00	.70643E-01	.64540E-01	.73918E-01	.36091E-01	9205E-0	3698E-
٥	r	20692E+02	21656E+02	18524E+02	.24495E+01	22331E+02	19395E+02	.79365E+00	21748E+02	19534E+02	.16448E+01	20589E+02	18364E+02	.29155E+01	21070E+02	17800E+02	.28118E+01			⋖		22964E+02	22656E+02	21512E+02	21637E+02	47952E-01	24337E+02	24337E+02	36402E+01	23768E+02	23781E+D2	28989E+01	23197E+02	23184E+02	20525E+01	22693E+02	22782E+02	10306E+01
ж ХДН Х.О		195.400	00000	0.000	5.700	3.900	3.900	5.500	5.400	5.400	5.400	7.500	7.500	7.500	6.800	90.800	182.700		MAX	RHDR	GH/CH2	5.400	195.400	0.000	000.0	000.0	3.900	3.900	3.900	5.400	5.400	5.400	2.500	7.500	7.500	96.800	₽•800	82.700
X O	GH/CH2	146.500	77.770	77.770	77.770	4.149	4.149	10.690	13.370	13.370	19.220	32.110	32,110	32,110	47.910	47.910	53.350	HHAS	HIN	RHOR	GH/CH2	146.500	46.5	7.17	.77	7 • 7 7	• 1.4	• 7 4	69.0	3.37	3.37	9.22	32.110	2.11	2-11	7.91	7.91	• 91
NEUTRONS PLING	DOSE	SILICON	~	TISSUE	FLUENCE	SILICON	TISSUE	FLUENCE	SILICON	TISSUE	FLUENCE	SILICON	TISSUE	FLUENCE	SILICON	TISSUE	FLUENCE	SECONDARY GAHMAS	ن	w	DOSE	SILICON	TISSUE	SIFICON	TISSUE	FLUENCE	SILICON	TISSUE	FLUENCE	SILICON	TISSUE	FLUENCE	SILICON	TISSUE	FLUENCE	_	ISSUE	FLUENCE
NEUTRO SAMPLING	(KM)	12.03	2 6 7	6.85	4.85	0.61	0.61	10.0	2.08	2.08	2.08	6.13	26.13	6.13	4.72	4.72	4.72	SECO	SAMPLIN	ALTITUD	(KH)	• 09	60	.85	85	•85	• 61	•61	• 61	• 08	• 08	80.	.13	.13	13	- 72	•	•

TABLE 18

FIT COEFFICIENTS FOR MORSAIR RUN NO. 12 THERMONUCLEAR SOURCE IN REAL AIR AT 30 km

RMS PCT DIFF	1.218	.275 .430 .342	. 321 . 321 . 211	.487 .323 .648	RMS PCT DIFF	.313 .303 .260	.308	.226	944·
ی	••••		• • • • •		v		,,,,,		••••
u.	••••		• • • •	•••••	u.		99387E+01 10320E+02 77550E+01		• • • • •
M		00	00		w y	00000			0000
FIT COEFFICIENTS 0	0. 0. 0.98193E-02			57490E-02 .14176E-01 .10404E-01 .11667E-01 .55760E-01 .51918E-01	T COEFFICIENT 0	0. 0. 0. -04 -17383E-03 -04 -80026E-03		28526E-02 28526E-02 11528E-01 19928E-01	
S I	0. 0. 25685E-03	86811E-04 72538E-04 59160E-04	21156E-03 58992E-03 .10934E-02 .89544E-03	.3654E-03 74157E-03 46226E-03 28823E-02 25667E-02	C FIT	0. 0. 30690E .56505E	-11514E -11514E -89892E	.24293E-04 .24293E-04 .43853E-03 .10059E-02	
83	1111	iii	• • • •	20074E-01 11752E+00 12071E+00 33355E+00 31538E+00	œ	1 1 1 1 1	3 1 1		1 1 1
ব	21861E+02 19699E+02 .11590E+01	18674E+02 23105E+01	20938E+02 13716E+01 22856E+02 19958E+02	-14873E+01 -21409E+02 -18873E+02 -22202E+01 -20106E+02 -17608E+02	∢	22308E+02 22538E+02 16965E+00 22112E+02	11598E+41 23441E+02 2333E+02 38715E+01	23177E+02 23491E+01 23436E+02 23436E+02	24581E+01 22059E+02 22081E+02 57843E+00
MAX RHOR GM/CM2	175.000 175.000 175.000	175.000 175.000 200.000	200.000 168.400 156.000 156.000	170,400 143,700 136,000 143,700 105,600 1119,000	MAX RHOR GH/CH2			80.000 80.000 80.000 143.700	9699
MIN RHOR GH/CH2	114.200 114.200 114.200	46.240 46.240 1.755	1.755 12.200 11.040 11.040	0 4 0 2 0 0 2 0 0 3 0 0 8 0 0 8 0 0	HHAS HIN RHOR GH/CH2	6.240 6.240 6.240	77. 77. 77. 70. 70. 70.	11.040 11.040 14.550 13.200	N 80 80 N
SAMPLING ALTITUDE (KH) DOSE	0000	300	2222	40.00 FLUENCE 50.00 SILICON 50.00 FLUENCE 60.00 SILICON 60.00 TISSUE 60.00 FLUENCE	SECONDARY GAMMAS SAMPLING ALIIUDE (KM) 00SE GM/	00.00			

The second secon

TABLE 19

FIT COEFFICIENTS FOR MORSAIR RUN NO. 13 THERMONUCLEAR SOURCE IN REAL AIR AT 40 km

	RMS PCI DIFF	. 460	.193	9	.164	.108	.198	123	. 114	4	.072			RHS	PCT	1110	.296	776 •	364	.366		.255	162.	.355	.347	!	. 234	• 229	74.	169		
	IJ	• • • •						•			• •				ၒၟ		• •		• •			•	•	• •	• •	••	•			• •		•
	և	00.			• 0	0.00	0.0	•	• • • • • • • • • • • • • • • • • • • •		• •		•		և		•0	•	•			.10692E+02	7004E+01	v			•	•	•	• •		;
	w	. 0	• • • • • •		18905E+00 .43923E+00				• •		• c			**	ш			•	•		• •	.11443E+02	.10629E+02	•			0.		•	•		;
	FIT COEFFICIENTS D		25323E-01 19933E-01	317E	21043E-01 66707E-02	.30704E-01	16935E-02	552E	.16669E-01 10901E-01	.44022E-03	339E	766		COEFFICIENTS			0.	•0•	700	- 28232F-01	9	Ö	.12660E+00	.19570E+00	-11513E+00	13677E+00	75654E-01	77075E-01	ò٠	64362E-U1	9 9	•
	S FI	.15031E-03 .91314E-04	.14240E-02	.74588E-03	.85599E-03	73868E-03	58518E-03	20427E-04	22222E-U2 .95978E-03	.12514E-02	31374E-02	10934E-02		FIT	ပ		3 60 E	•4 29 32 E-04	.20713E-04	70-33724	.14374E-02	47785E-02	44242E-02	70956E-02	.89397E-02	-10689E-01	.64641E-02	•65680E-02	26E-0	.60704E-02	10001	0/ 90E-0
	83	396E-01 357E-01	566E-01 566E-01 364E-01	7437E-01		1975E+00		13048+00		4459E+00	81768E-01				63		28237E-01	377E-01	263E-01		4095+00		565E+01	587E+01	36520E+00	.42273E+00		.20573E+00	9095E+0	.14126E+00	4/2/610	331 4E4
	æ	21241E+02 18954E+02	22307E+02	.115736+01	22234E+02 20453E+02	87887E+00	22252E+U2 19711E+02	.12079E+01	22059E+02 19587E+02	.13427E+01	22065E+02	-13515E+01			Ø		22171E+02	22220E+02	67739E+00	Z44435+84	31934E+01	24992E+02	25263E+02	31969E+01		35257E+01	23866E+02	23905E+02	27478E+01	23639E+02	236/86+02	651606+U1
	HAX RHOR GH/CH2	9.30	ກໍເຄີຍ	• •	76.530	٥٥	42.630	å	32.420	ż		24.550	٠	7	RHDR	GM/CM2		179.300	179.300	120.600	120.600	76.530	76.530	76.530	46.630	42.630	32.420	32.420	32.420	24.550	74.550	ncc • + 7
	HIN RHOR GH/CH2	54.830	10.110	11.610	.750	.750	3.445	4.985	3.661	3.661	3.674	3.674	HHAS	2	RHOR	GN/CH2	54.830	E4.830	54.830	16.110	11.610	.750	.750	.750	0.445V	4.985	3.661	3.661	5.160	3.674	3.674	3.6/4
NEUTRONS	SAMPLING ALTITUDE (KM) DOSE	0.00 SILICO 0.00 TISSUE	20.00 FLUENCE 30.00 SILICON	00.0	0.00 SILICO	00.0	-	0.00		80.00	0.00	80	SECONDARY GAHHAS	0740	TOE TOE	(KH) DOSE	0.0	00.	20.00 FLUENCE	80.		OD SITICO	00.	00.	60 00 TISSUE		00.	00.	• 00	0	00.00	100.00 FLUENCE

TABLE 20

FIT COEFFICIENTS FOR MORSAIR RUN NO. 14 THERMONUCLEAR SOURCE IN REAL AIR AT 60 km

NEUTRONS

RMS	PC1 01FF	• 849 • 889	1445	. 162	1	840.	250.	620) •	970	770		• 030	•029			RMS	PCT	OIFF	• 454	. 472	ć	617.	• • • • • • • • • • • • • • • • • • • •	2	840		650.	• 058	0	600.		720	078) •		
,	ဖ	•••								•	•			0.	.0				ی		0.	0.	.0	•0	•	•	•	• •	•0	.0	•		•		•	• •		
	lı.	0.0	•0•	•	• •	. 0	0.	• 0	•	•	•	• =	• •			0.			u.		0.	g.	0.	•	•0	0.	20400E40C4	-38368E+01	.26355E+02	.31812E+02	.15363E+02	. 54357E+02	• 54485E +02	• 35 43 9E +UZ	. 13/1/6+03	2 5		
	ш	•••	0	9		10654E+0	•	.84360E+0		.26271E+U	.67420E+U		AGUNZE+U	27639E+0	.41108E+0	0		1 0	L.	I	•		0.			0.	.2558ZE+UZ	.24/91E+UC	11737E+02	17926E+02	.19808E+01	43540E+02	4386E+02	20243E+02	- 14134E+U3	47.47.41.03	201374767-	
FIT COEFFICIENTS	0	0.0	0.	.44101E-01	- 11001E+01	75026E+00	70468E+00	53998E+00	.30111E+00	.57293E+00	.23037E+01	.23158E+U1	. C6826E+U1	16576F+01	.20675E+01	88730E		EIT COFFEIGIENTS		.	0		•0	3689E+0	23268E+00	5610E+00	526E+01	2312+01	26455F+01		44032E+01		57E+00	.45960E+01	.12552E+02		*******	
FI	ပ	.97844E-04	70539E-05	61047E-02	27443E-U2 44644E-U2	.11838E+00	.11326E+00	.10526E+00	43473E-01	59545E-01	31576E+00	47536c+00	4/38/E+UU		.33453E+00	.18317E+0		114	•	,	.27690E-03	28204E-03	.14795E-03	.20442E-01	.19974E-01	.23313E-01	27014E+00	26457E+UU	- 28241 6+00	29106E+33	61399E+00	11881E+00	84035E-01	91 99E	8 59E	*	•19489E+01	
	æ	49680E-01	26746E-01	12772E+00	.57673E-UI	165485400				20199E+01	.62.934E+01	.42868E+01	.57370E+01		46415F+01	15879E+02			c	a	704865-01	72210F-01	.47585E-01					10376E+02		34919E+01	11050E+02	.31458E+01	v	3613E+D	546E+0	2E+	8 38 UE + U	
	⋖	21896E+02	.11895E+01	22189E+02	19970E+02	.49316E+UU	20082E+02	35342E+00	22772E+02	20985E+02	19236E+01	23275E+02	21441E+02	20691E+01	C3U55E+UC	23554E+01			<	₹	2000	2007	137	51	25195E+02		-,28596E+02	28725E+02	20183E+05	36086F+02					87.	52975E+02	806	
×ΦΗ	RHOR GH/CH2	108.100	စ်ဆိ	25.990	2 • 930	٠,	• •	9.715		•	•	•	3.451	•	2 4	2.484			YYU	KHUK GH/CH2	٥				26.990	å	9.715	9,715	9.(15	5.139	5.139	3.451	3.451	3.451	•		48	
2	RHOR GM/CH2	062.90	7.90	2.889	88	619	054.	12	31	.31	.31	•28	•28	.28	9 0	.287	GAMNAS	;	NIN	KHUK GM/CM2	60	067.4	a =	2,889	88	.19	.150	.150	Α:	0 T C	315	28	.286	28	.287	.287	.287	
NEO INC.	ALTITUDE (KM) DOSE	IS DO.	ગ ગ	OD SIFICO	• 00	٠,	2 C		000	00.	86.00	00.	100.00 TISSUE	2	OO SIL	120.00 fluence	SECONDARY GAN		SAMPLING	ALTITUDE (KN) DOSE	6	200) C			00.0	00.0	60.00 TISSUE	00.00	• •	00.00	00	00.	. 00 F	.00 SILIC	O TISSUE	٩.	

TABLE 21

FIT COEFFICIENTS FOR MORSAIR RUN NO. 15 THERMONUCLEAR SOURCE IN REAL AIR AT 80 km

	RMS PCT DIFF	1.022	.200	.076 .066	.035	.016	.007	.008		RMS PCT DIFF	.427	.346	348	.133	.060	• 929	. 089 090	• 062	.064	.002	.003	
	ဖ	.00		000	• • • • • • • • • • • • • • • • • • • •	• • • • •	• • • • • • • • • • • • • • • • • • • •	••••		u	00.	• • •	• •		•0			• •	0.	• 0	0.0	
	L			000			000	••••		u,	.	• •	0.0	.10329E+03 .93335E+02	-11078E+03 74238E+01	-,79054E+01	.65207E+02	.95865E+02 .38323E+03	.39526E+03	.57147E+03	.14039E+04	
	S E							.22984E+02 .90426E+00 .75787E+01 .25308E+02		ίν m	000	9 0					35236E+02 43202E+02					
	FIT COEFFICIENTS D	0. 0. 0. 1.447696+00						.13645E+03 41709E+02 .75289E+01		FIT COEFFICIENTS O							.22720E+03				64310E+04 57965E+04	
	ũ υ	32059E-02 61874E-03 21110E-02	.17602E-01	.11156E+01 .56572E+00	.48434E-U1 93985E+00 .43925E+01	.13115E+02 .29707E+02 .33683F+02	.29885E+02 .37755E+02 40514E+02	65230E+02 .50700E+02 .18617E+02 13722E+03			42259E-02 72011E-02						13574E+03 12430E+03				.46433E+0)))
	33	36483E+0 28102E-0 17868E+0	23108E+0 52952E+0	79842E+0 29896E+0	34827E+U1 66020E+U0 .55613E+U1	12961E+0 13079E+0 72731F+0	.15078E+0 .31382E+0	90866E+02 .77458E+01 21028E+02 11692E+03		œ	. 50399E+00 . 86602E+00			•	i	1 1	ří	ľ		•		1
	4	34946E 20985E 36652E	0025E	36.8		65852E-01 22365E+02 20400F+02	i iii iii jii	14574E+01 22477E+02 20617E+02 13697E+01		đ	39011E+02 50159E+02	24993E+02	24972E+02 41507E+01	-,45474E+U2 -,44115E+U2	26118E+02 31936E+02	31900E+02	36705E+02	17149E+02 53629E+02	54207E+02	41869E+02	10379E+03	1
	MAK RHOR GH/CH2	64.860 64.860 63.500		mm		. 832 . 289	. 226 . 153			HAX RHOR GH/CH2	64.860 64.860	12.900	12.900 12.900	3.404 3.404	3.404	. 832	.289	. 289	.153	. 153	111	[]
	HIN RHOR GH/CH2	000) O @	25.	$\omega - \omega$.020	1 4 4 4	.015 .015 .015	IHAS	HIN RHOR GH/CH2	തത	3.00	3,008	.267	.267	200	.015	10	.015	50	100)
NEUTRONS	SAMPLING ALTITUDE (KH) DOSE	0000		00.00	000	0.00	120.00	00000	SECONDARY GAHHAS	SAMPLING ALTITUDE (KH) DOSE	000	90	00	000	000	8 8	.00 SILICO	00	.00 TISSUE	00 FL		

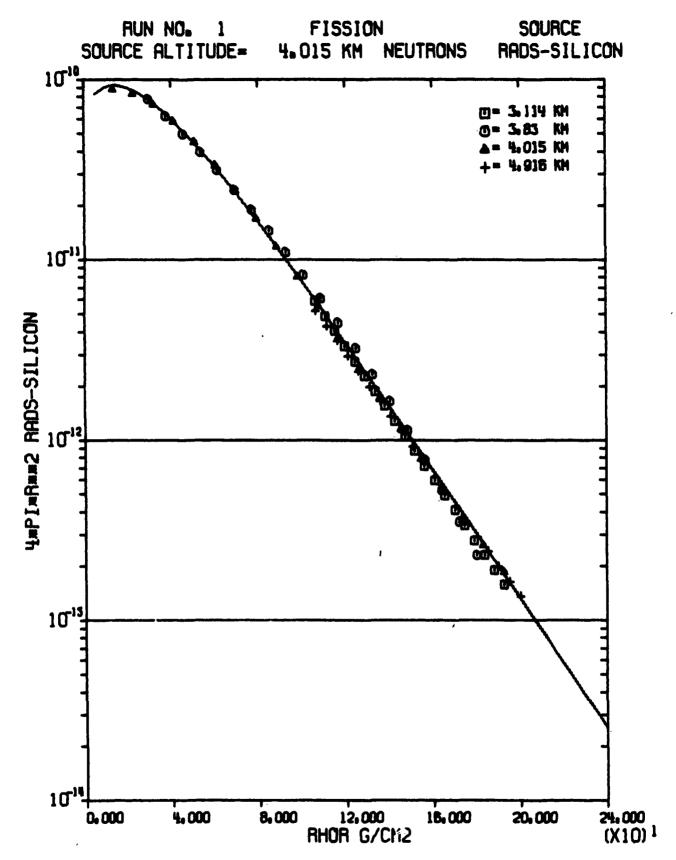
APPENDIX C

MORSAIR SILICON DOSES AND K-FACTORS

For each of the 15 MORSAIR runs, this appendix contains the computer plotted results of:

- 1. The fit of the MORSAIR $4\pi\,R^2$ silicon dose versus the mass range ("RHOR" in the figures). The different point symbols were calculated from the fits to the MORSAIR data, and the solid line is the fit of the one-dimensional ANISN data.
- 2. The silicon dose K-factors as a function of mass range at each sampling altitude.

In order to reduce the number of plots, several sampling altitudes have been combined on a single figure.



FIGUR_ C- 1 MORSAIN FIT DATA-4PIR**2 NEUTRON SI DOSE, FISSION SOURCE IN HOMO AIR AT 4.8 KM. ALL SAMPLING ALTITUDES

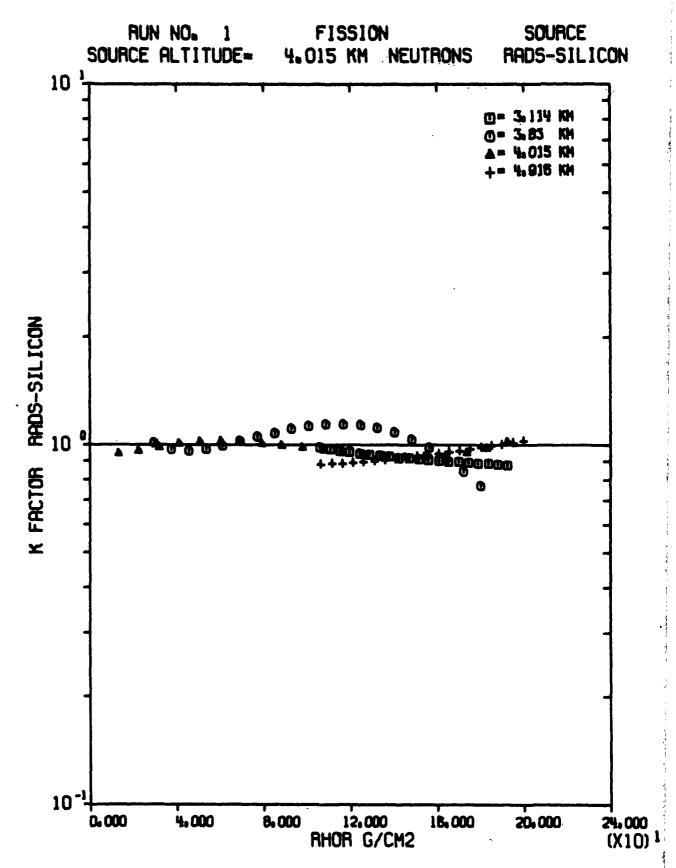


FIGURE 6- 2 MORSAIR FIT DATA-NEUTRON SILICON K-FACTOR FIGSION SOURCE IN HOMO AIR AT 4.0 KM. ALL JAMPLING ALTITUDES

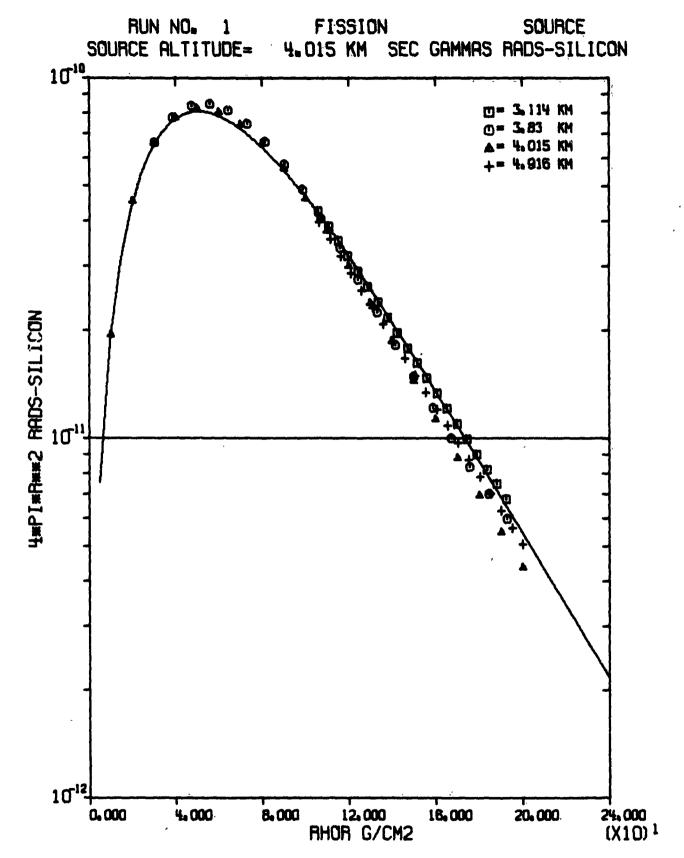


FIGURE 3- 3 MORSAIR FIT DATA-4PIR+42 GAMMA SI DOSE, FISSION SOURCE IN HOMO AIR AT 4.0 KM. ALL SAMPLING ALTITUDES

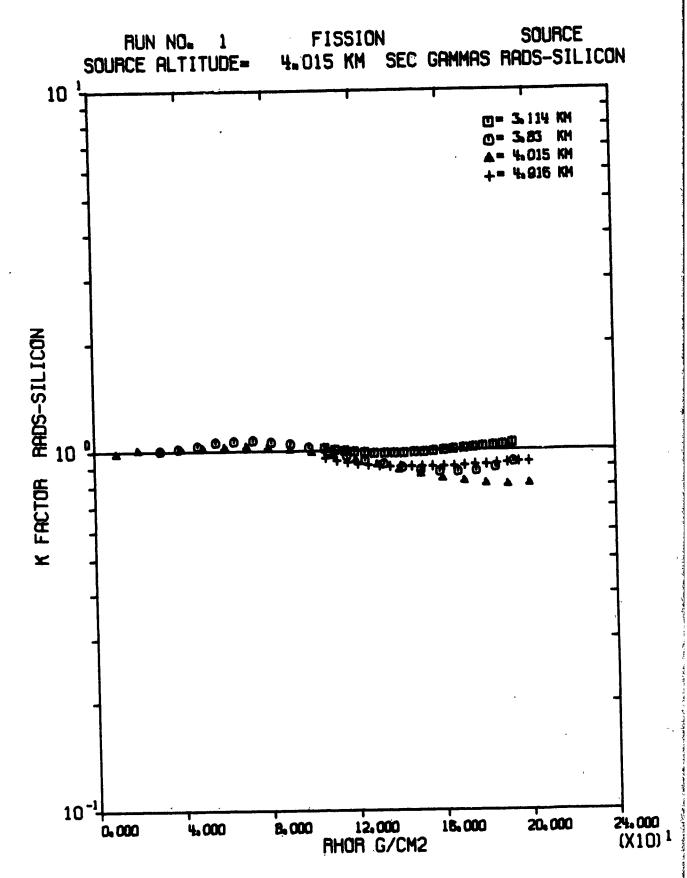


FIGURE 0- 4 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR, FISSION SOURCE IN HOMO AIR AT 4.0 KM. ALL SAMPLING ALTITUDES

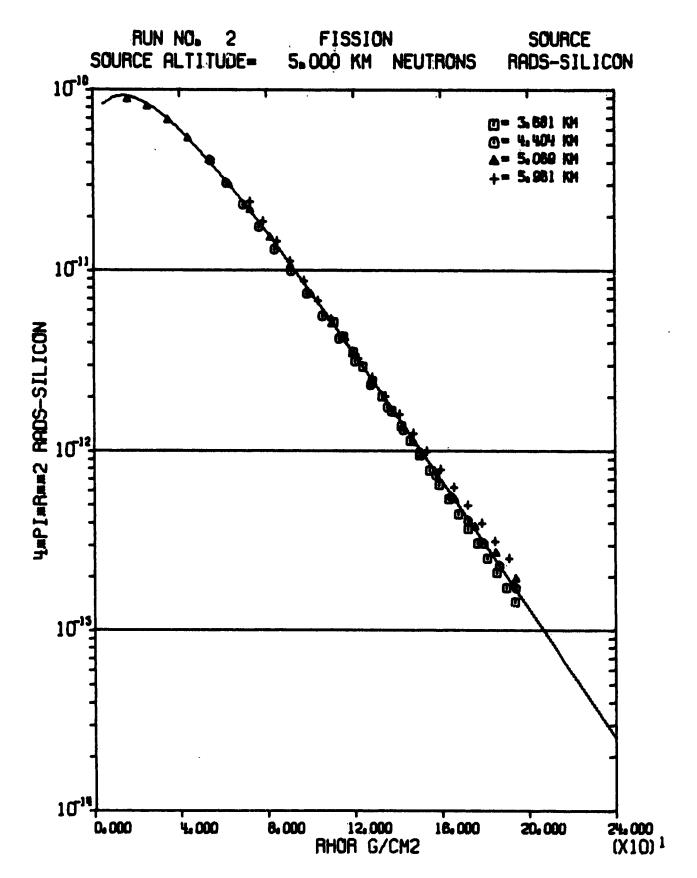


FIGURE C- 9 AURSAIR FIT DATA-4PIR**2 NEUTRON SI DOSE. FISSION SOURCE IN REAL AIR AT 5.0 KM. ALL SAMPLING ALTITUDES

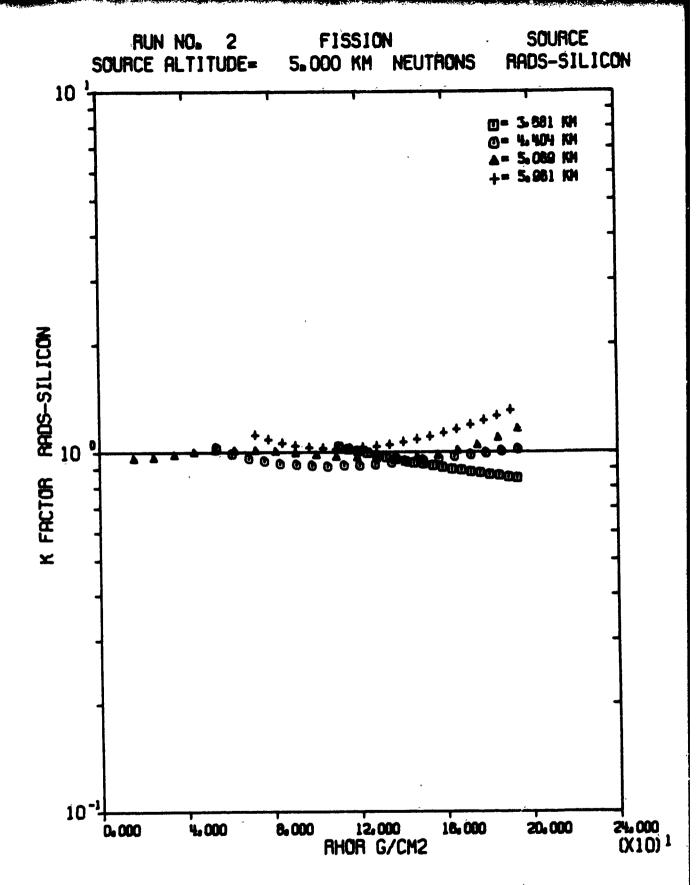


FIGURE C- 6 MORSAIR FIT DATA-NEUTRON SILICON K-FACTOR FISSION SOURCE IN REAL AIR AT 5.0 KM. ALL SAMPLING ALTITUDES

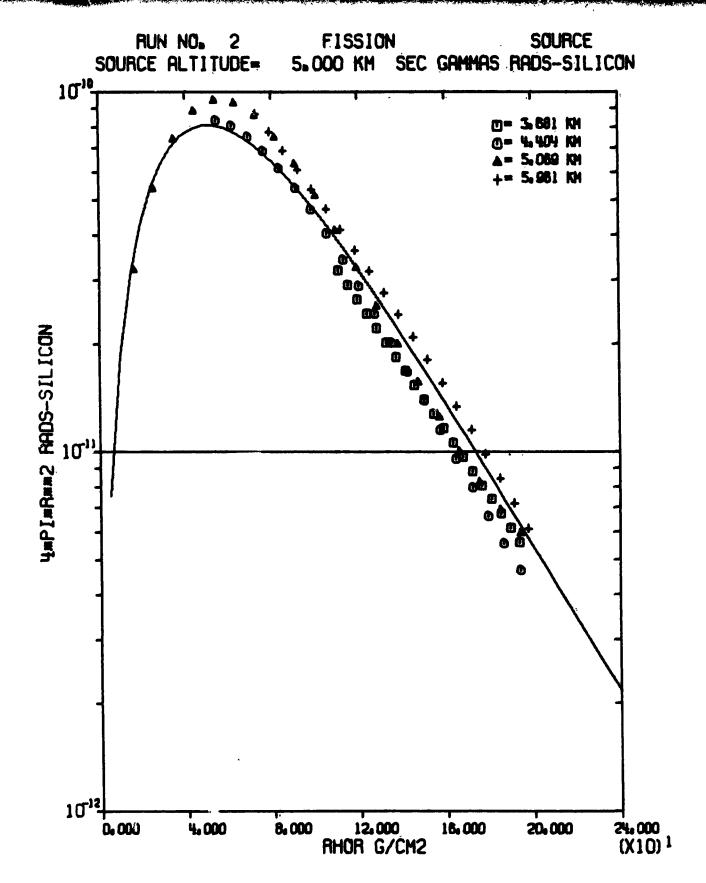


FIGURE G- 7 MORSAIR FIF DATA-4PIR**2 GAMMA SI DOSE.
FISSION SOURCE IN REAL AIR AT 5.0 KM.
ALL SAMPLING ALTITUDES

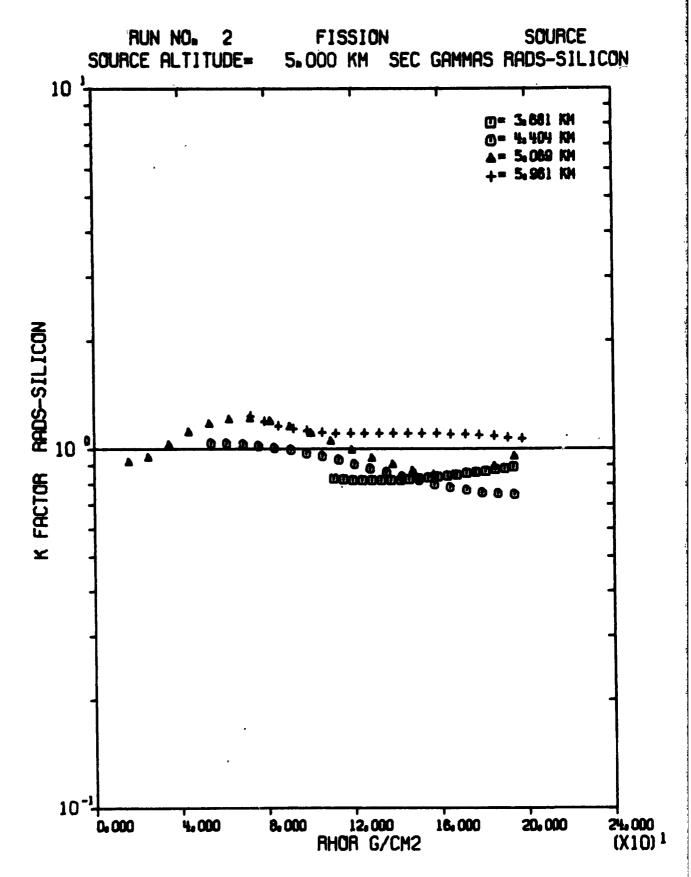


FIGURE C- 6 MORSAIR'FIT DATA-GAMMA SILICON K-FACTOR, FISSION SOURCE IN REAL AIR AT 5.0 KM. ALL SAMPLING ALTITUDES

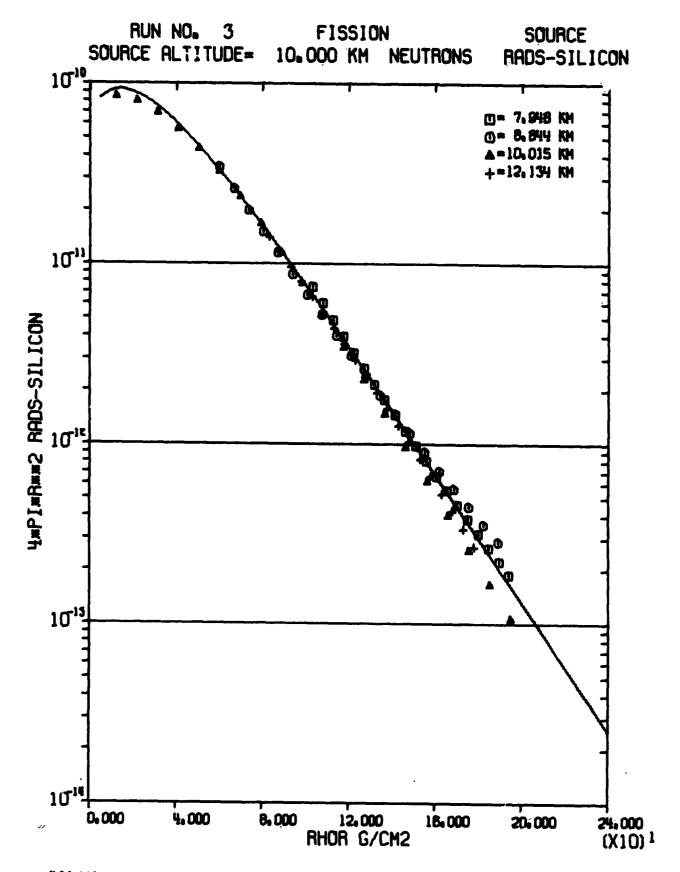


FIGURE U- 9 MORSAIP FIT DATA-4PIR**2 NEUTRON SI DOSE, FISSION SOURCE IN REAL AIR AT 10.0 KM. ALL SAMPLING ALTITUDES

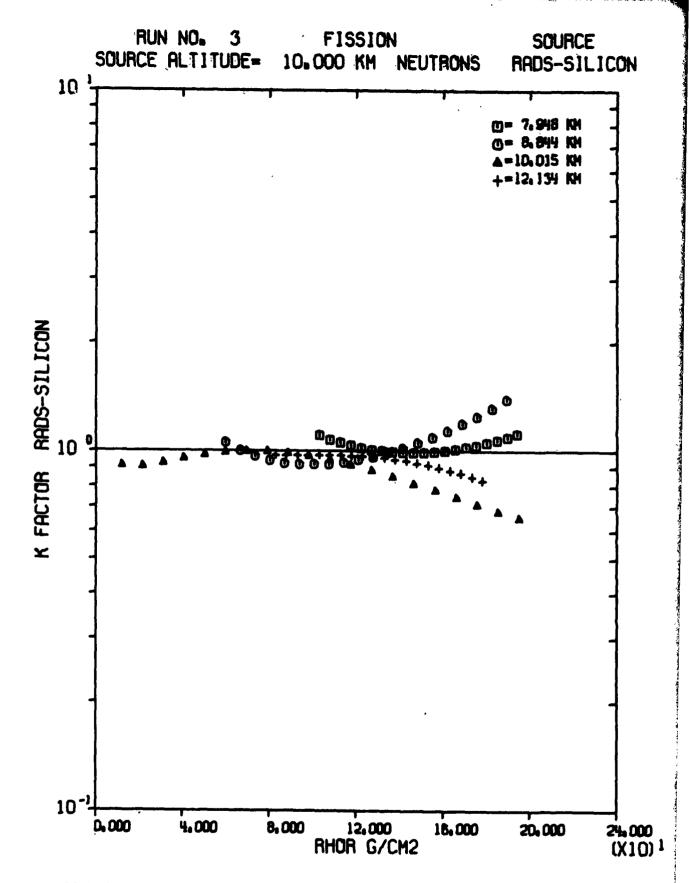


FIGURE C-10 MURSAIR FIT DATA-NEUTRON SILICON K-FACTOR FISSION SOURCE IN REAL AIR AT 10.0 KM. ALL SAMPLING ALTITUDES

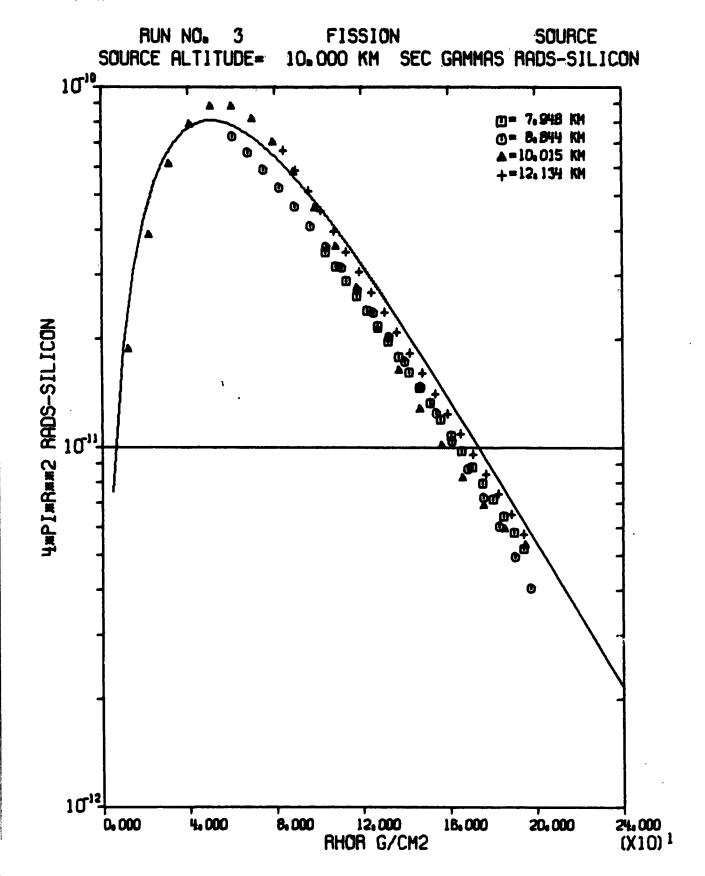


FIGURE 0-11 MORSAIR FIT DATA-4PIR##2 GAMMA SI DOSE.
FISSION SOURCE IN REAL AIR AT 10.0 KM.
HUL JAPPLING ALTITUDES

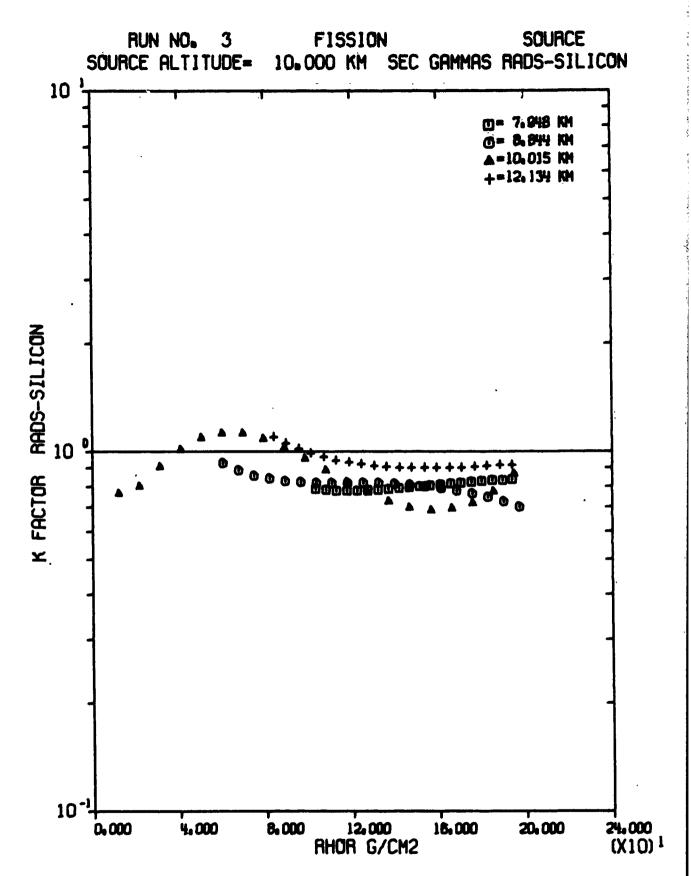


FIGURE C-12 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR.
FISSION SOURCE IN REAL AIR AT 10.0 KM.
ALL SAMPLING ALTITUDES

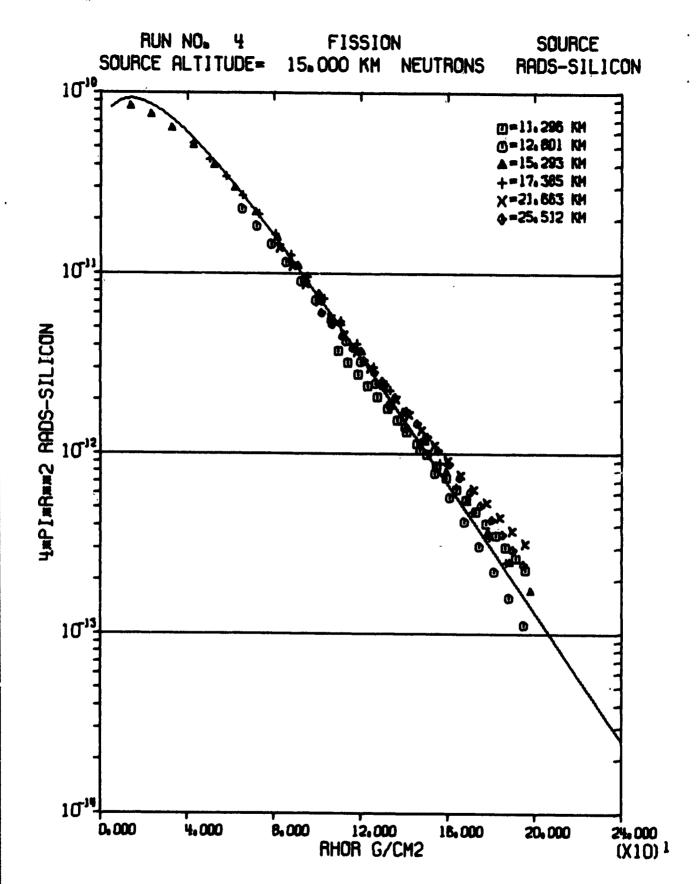


FIGURE C-13 MORSAIR FIT DATA-4P1R**2 NEUTRON SI JOSE., FISSION SOURCE IN REAL AIR AT 15.0 KM. ALL SAMPLING ALTITUDES

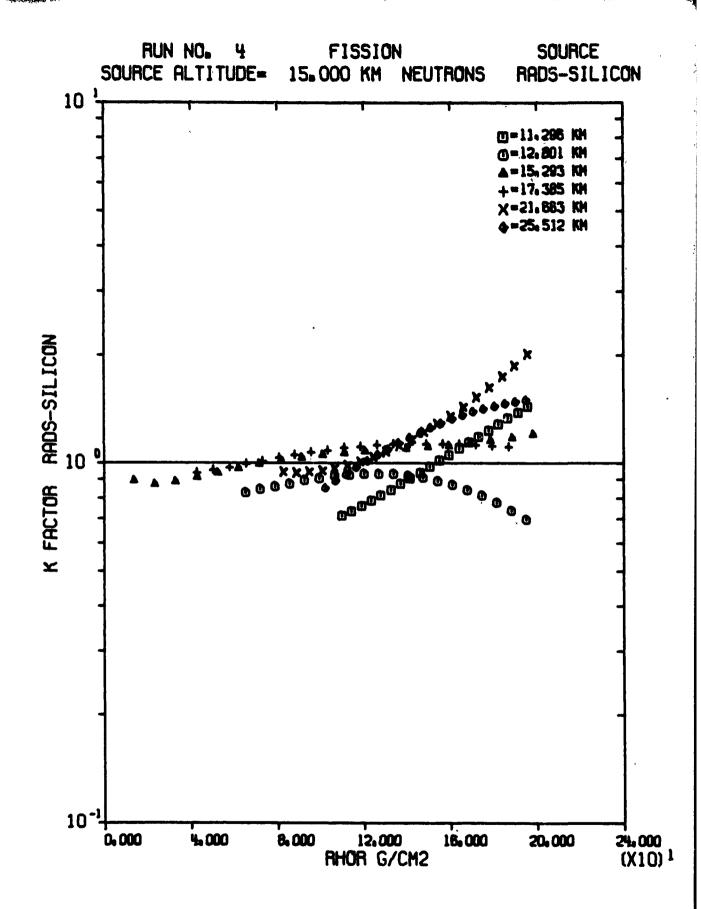


FIGURE C-14 MORSAIR FIT DATA-NEUTRON SILICON K-FACTOR FISSION SOURCE IN REAL AIR AT 15.0 KM. ALL SAMPLING ALTITUDES

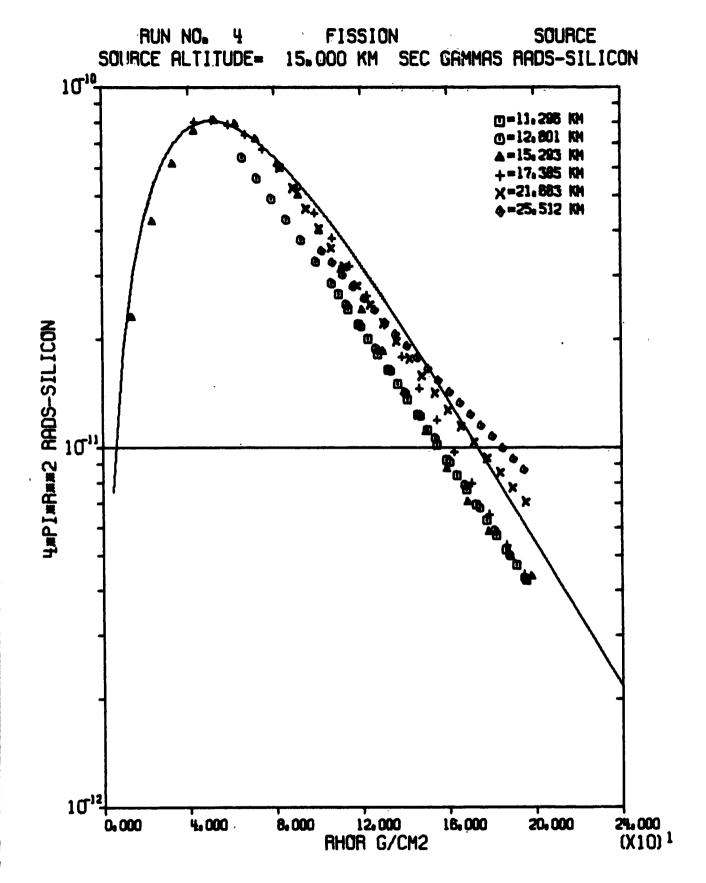


FIGURE 0-15 MORSAIR FIT DATA-4PIR**2 GAMMA SI DOSE. FISSION SOURCE IN REAL AIR AT 15.8 KM. ALL SAMPLING ALTITUDES

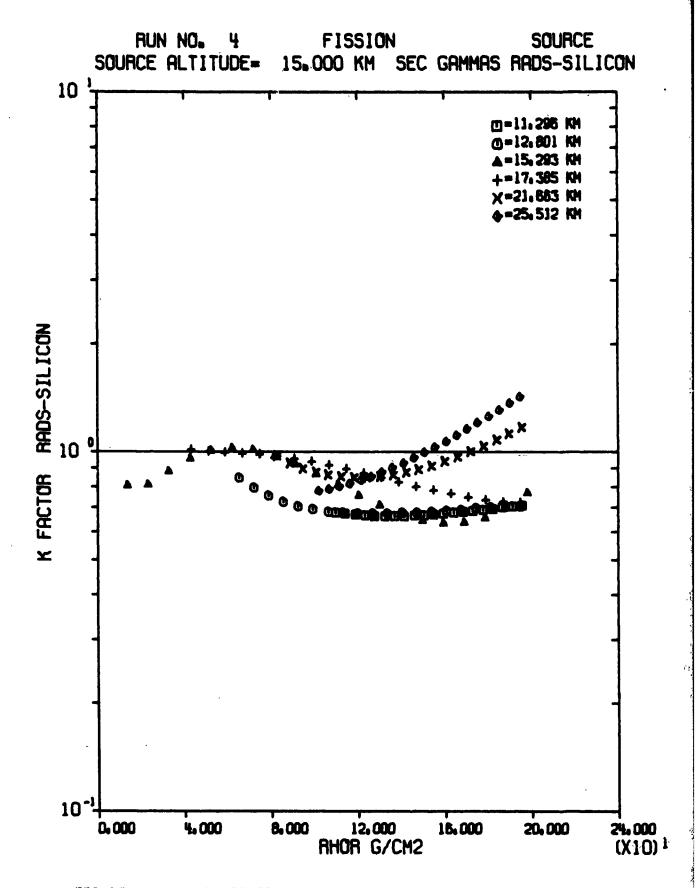


FIGURE C-16 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR.
FISSION SOURCE IN REAL AIR AT 15.0 KM.
ALL SAMPLING ALTITUDES

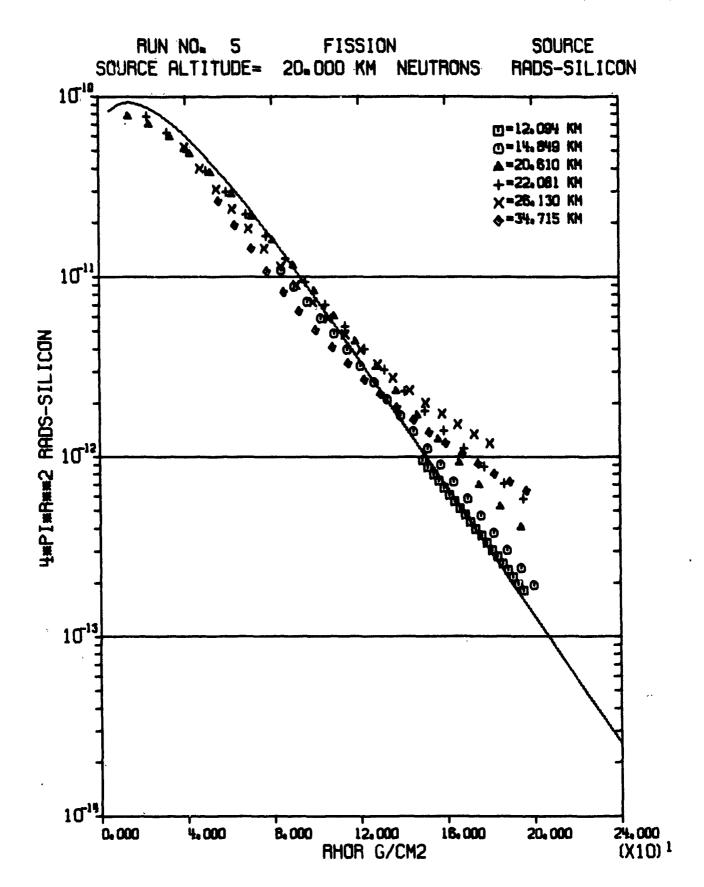


FIGURE C-17 HORSAIR FÍT DATA-4PIR#+2 NEUTRON SI DOSE, FISSION SOURCE IN REAL AIR AT 20.0 KM. ALL SAMPLING ALTITUDES

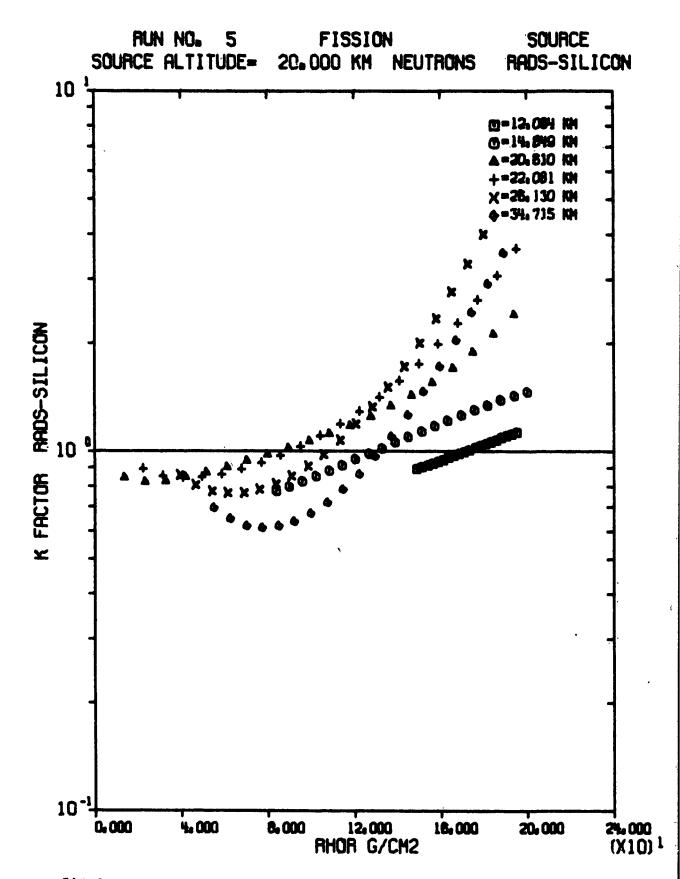


FIGURE C-18 MORSAIR FIT DATA-NEUTRON SILICON K-FACTOR FISSION SOURCE IN REAL AIR AT 28.3 KM. ALL SAMPLING ALTITUDES

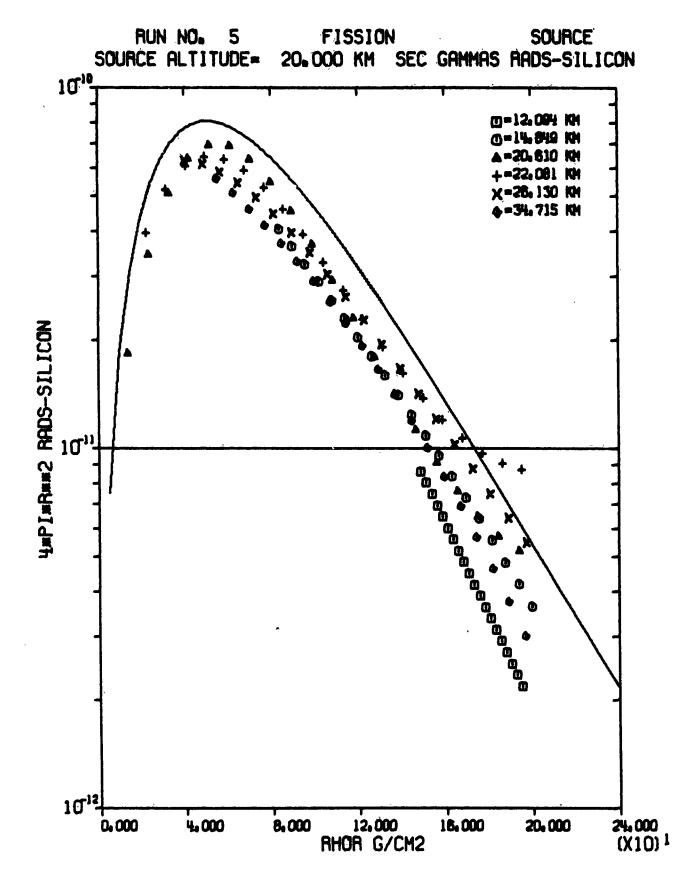


FIGURE C-19 HORSAIR FIT DATA-4PIR**2 GAMMA SÍ DOSE, FISSION SOURCE IN REAL AIR AT 20.0 KM. ALL SAMPLING ALTITUDES

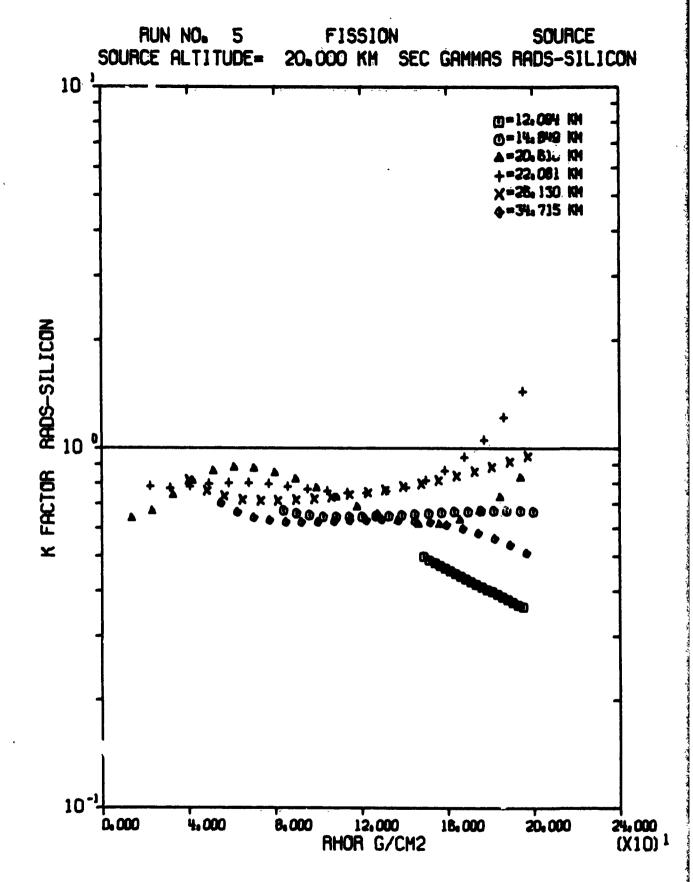


FIGURE C-20 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR.
FISSION SOURCE IN REAL AIR AT 20.0 KM.
ALL SAMPLING ALTITUDES

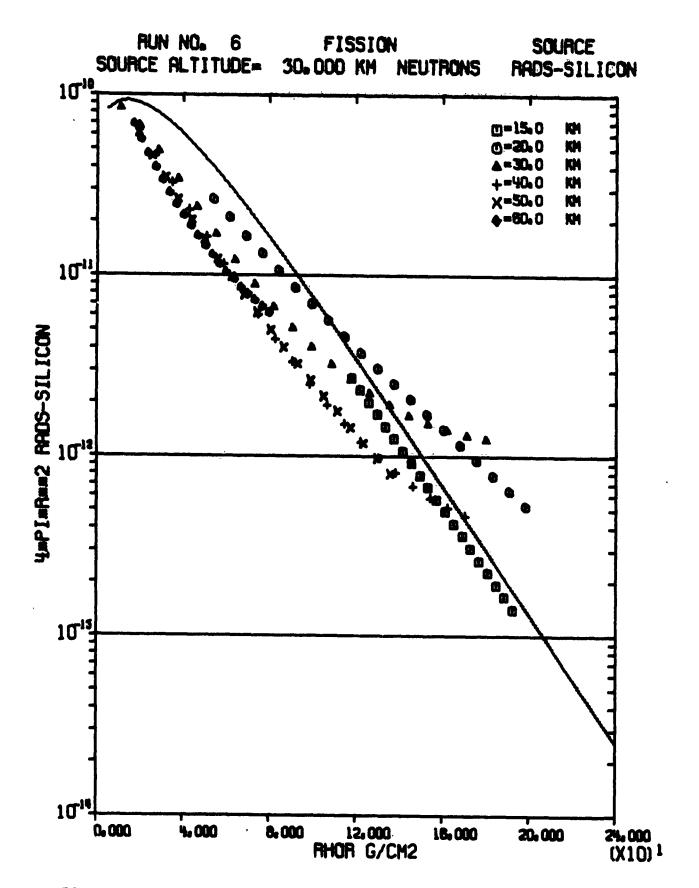


FIGURE C-21 MORSAIR FIT DATA-4PIR**2 NEUTRON SI DOSE, FISSION SOURCE IN REAL AIR AT 30.0 KM. ALL SAMPLING ALTITUDES

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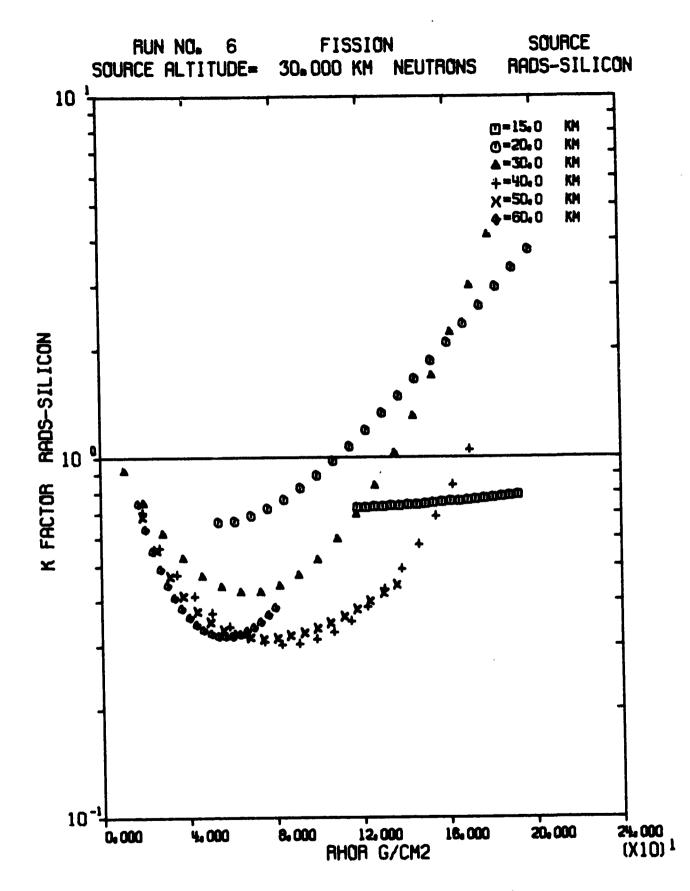


FIGURE C-22 MORSAIR FIT DATA-NEUTRON SILICON K-FACTOR FISSION SOURCE IN REAL AIR AT 30.0 KM. ALL JAMPLING ALTITUDES

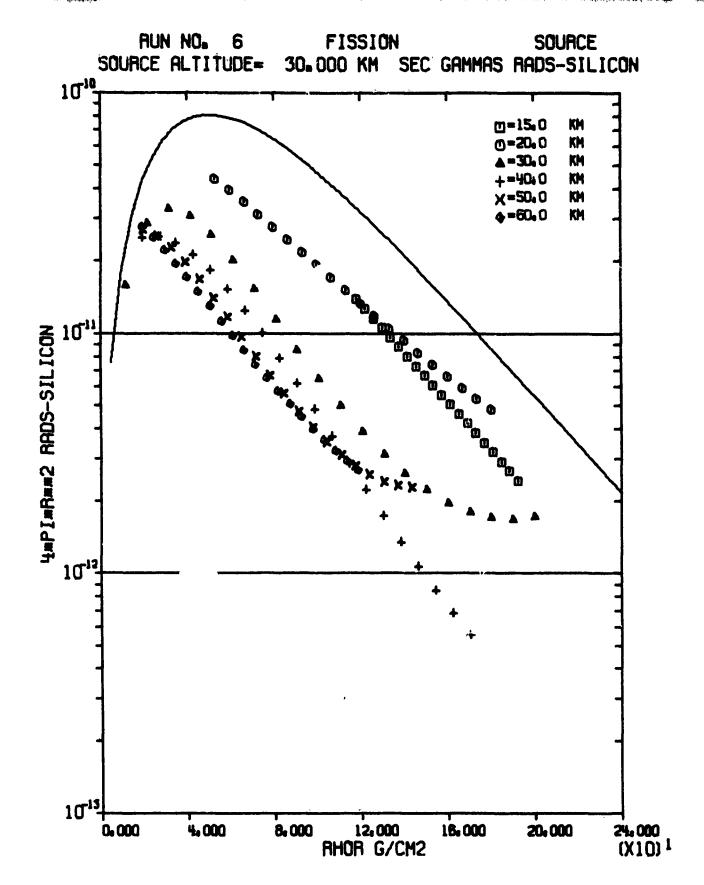


FIGURE 6-23 MORSAIR FIT DATA-RPIR**2 GAMMA SI DOSE, FISSION SOURCE IN REAL AIR AT 30.0 KM. ALL SAMPLING ALTITUDES

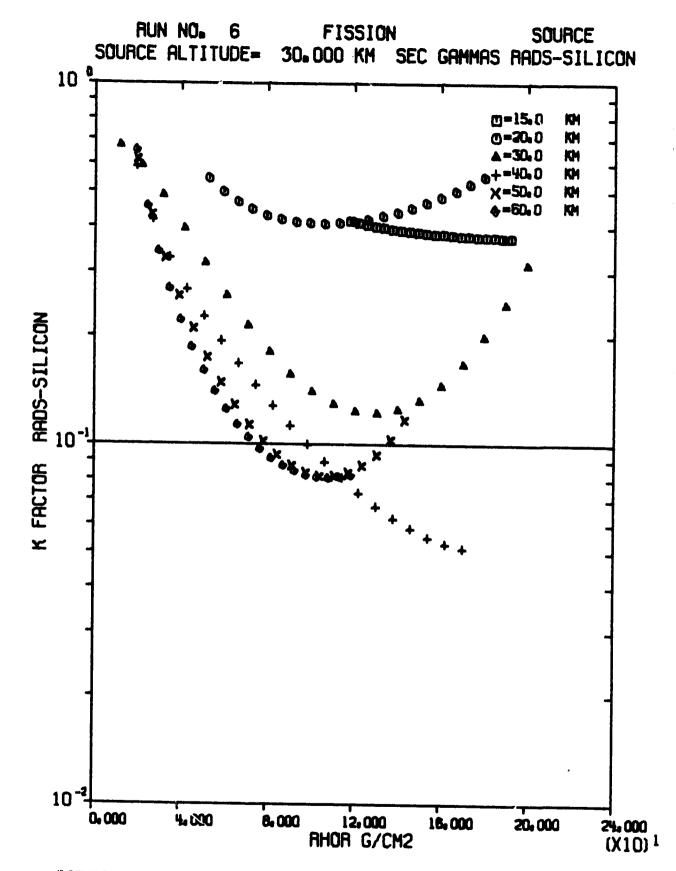


FIGURE C-24 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR, FISSION SOURCE IN REAL AIR AT 30.0 KM. ALL SAMPLING ALTITUSES

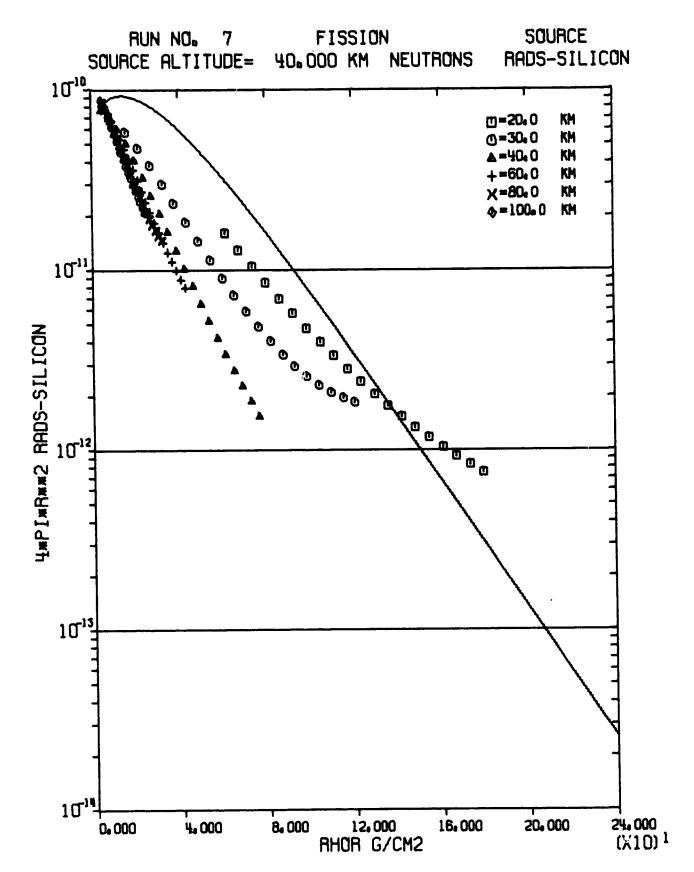


FIGURE 3-25 MORSAIR FIT DATA-4PIR**2 NEUTRON SI UOSE, FISSION SOURCE IN REAL AIR AT 40.0 KM. ALL SAMPLING ALTITUDES

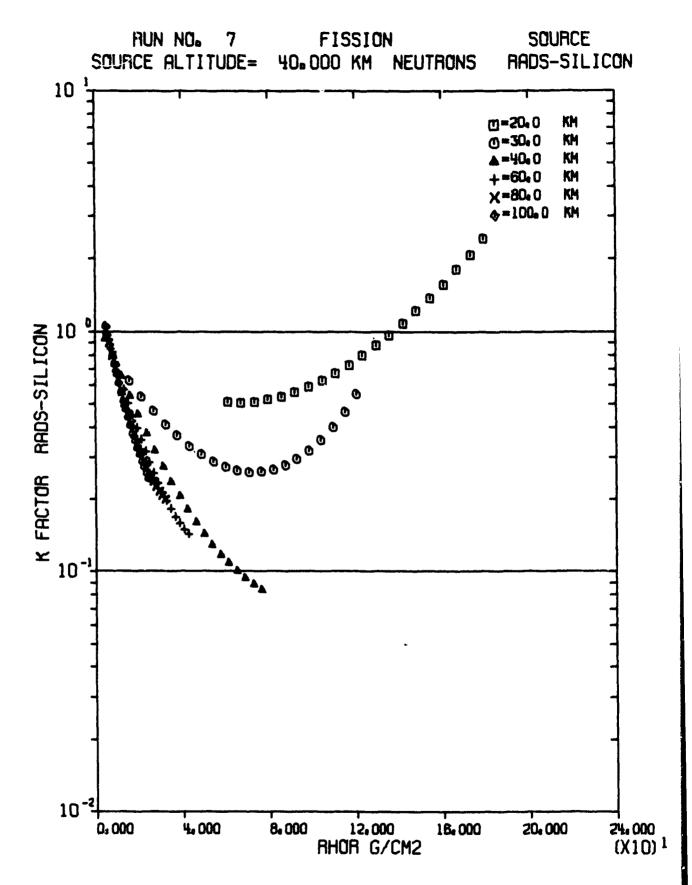


FIGURE 3-26 MORSAIR FIT DATA-NEUTRON SILICON K-FACTOR FISSION SOURCE IN REAL AIR AT 40.0 KM. ALL SAMPLING ALTITUDES

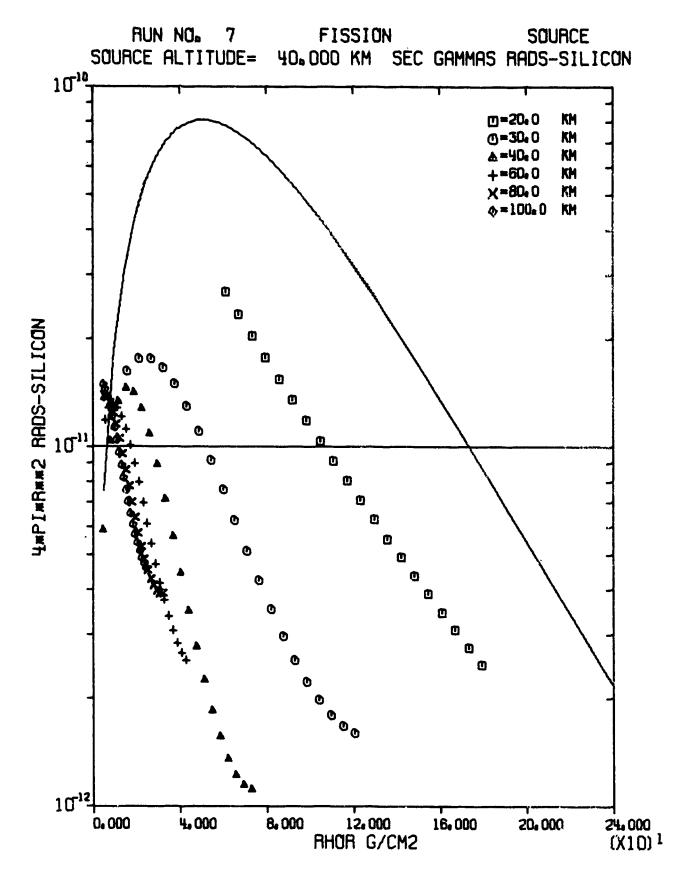


FIGURE 3-27 MORSAIR FIT DATA-4PIR**2 GAMMA SI DOSE. FISSION SOURCE IN REAL AIR AT 40.0 KM. ALL SAMPLING ALTITUDES

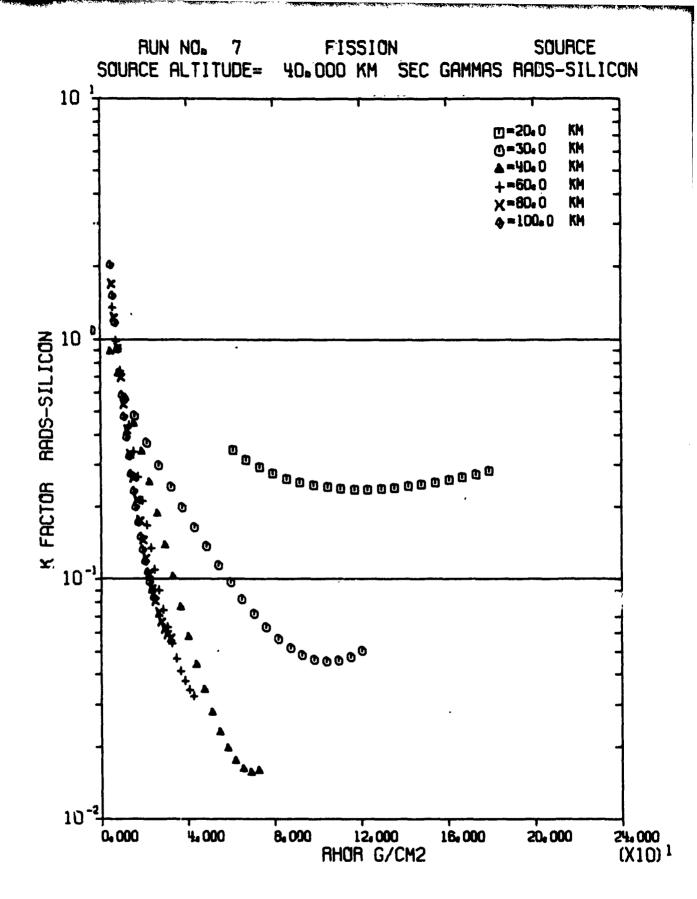
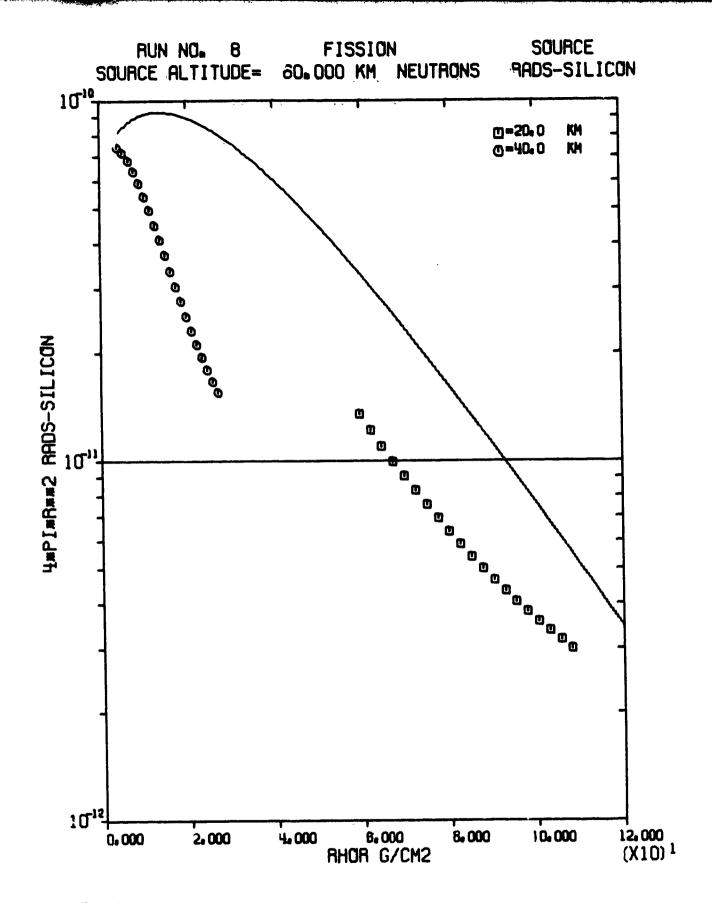


FIGURE C-28 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR, FISSION SOURCE IN REAL AIR AT 40.0 KM. ALL SAMPLING ALTITUDES



FIGURI C+29 MORSAIR FIT DATA-4PIR**2 NEUTRON SI DOSE, FISSION SOURCE IN REAL AIR AT 60.0 KM. SAMPLING ALTITUDES 20 AND 40 KM.

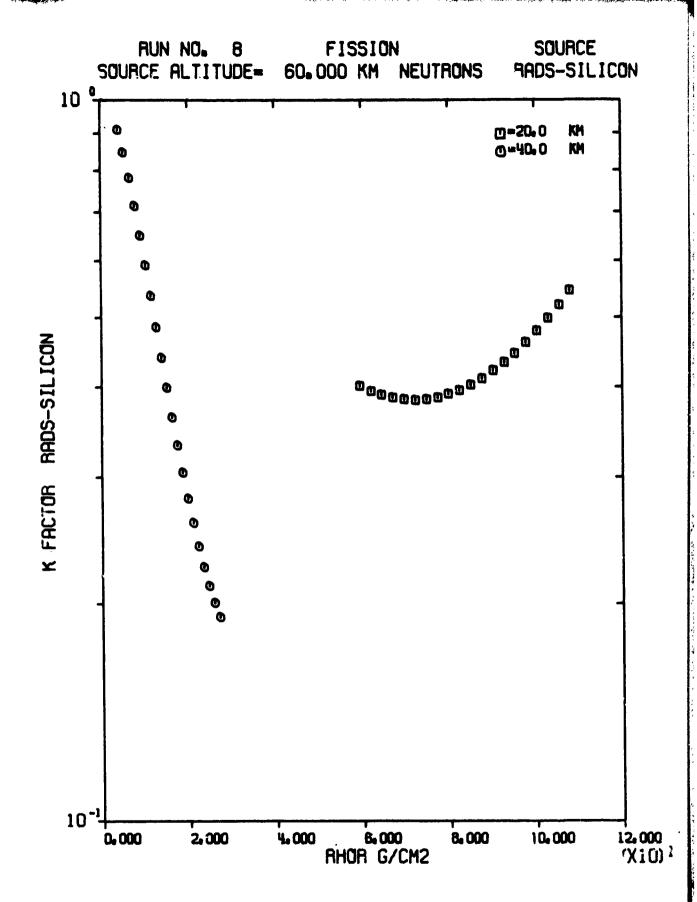
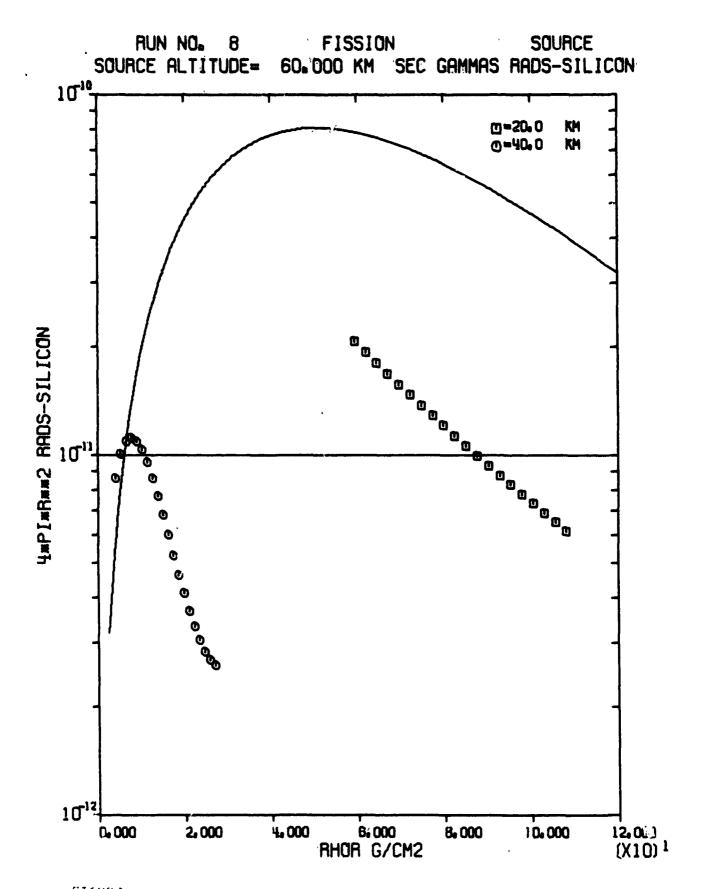


FIGURE C-30 MORSAIR FIT DATA-NEUTRON SILICON K-FACTOR FISSION SOURCE IN REAL AIR AT 60.0 KM. SAMPLING ALTITUDES 20 AND 40 KM.



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FIGURE .-31 JORSAIR FIT DATA-4PIR**2 GAMMA SI DOSE, FIJSION SOURCE IN REAL AIR AT 60.0 KM. DAMPLING ALTITUDES 20 AND 45 KM.

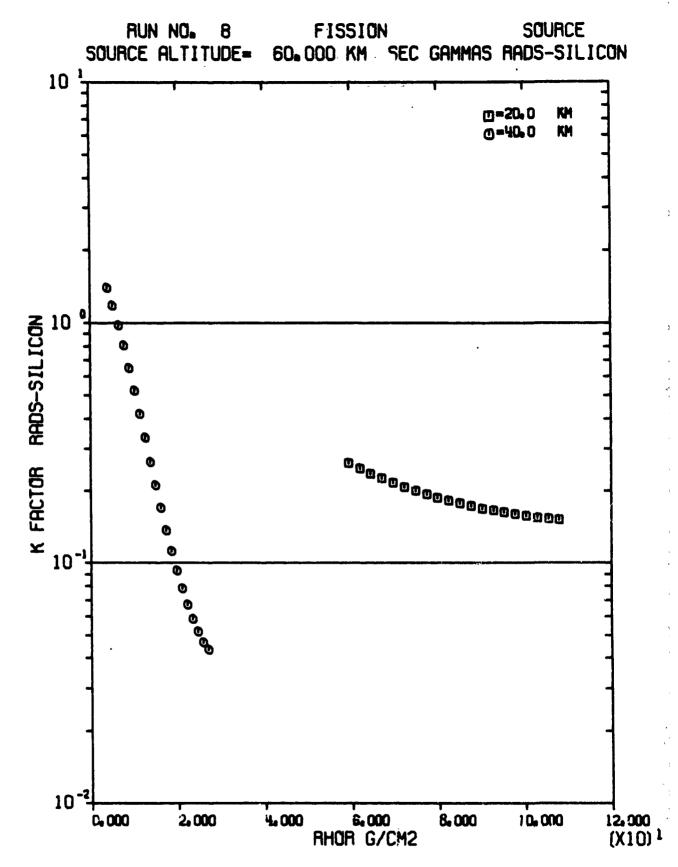


FIGURE C-32 MURSAIR FIT DATA-GAMMA SILICON K-FACTOR, FISSION SOURCE IN REAL AIR AT 60.0 KM. SAMPLING ALTITUDES 20 AND 40 KM.

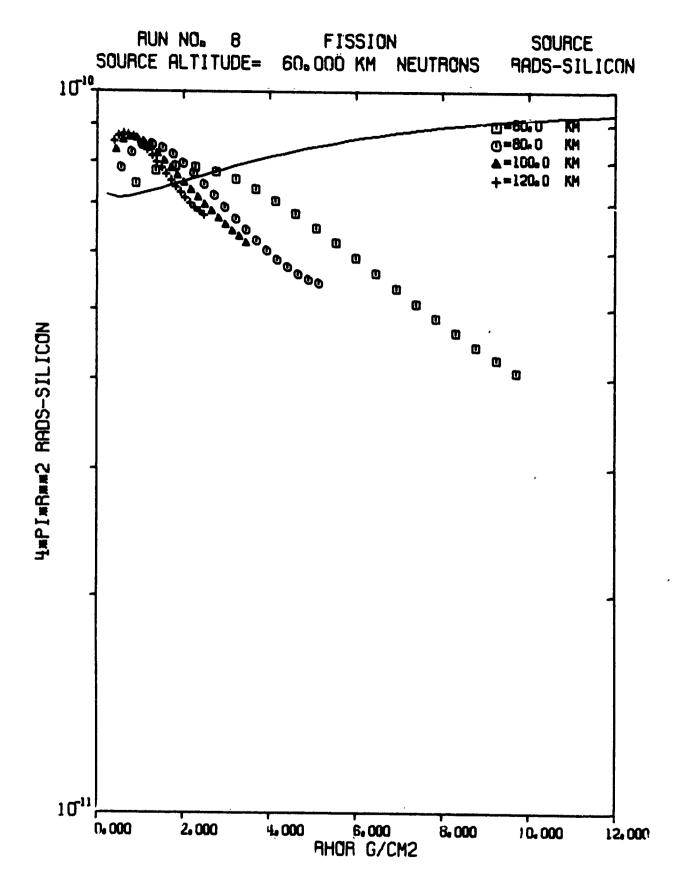


FIGURE C-33 MORSAIR FIT DATA-4PIR**2 NEUTRON SI DOSE, FISSION. SOURCE IN REAL AIR AT 60.8 KM. SAMPLING ALTITUDES 00.80,100, AND 120 KM.

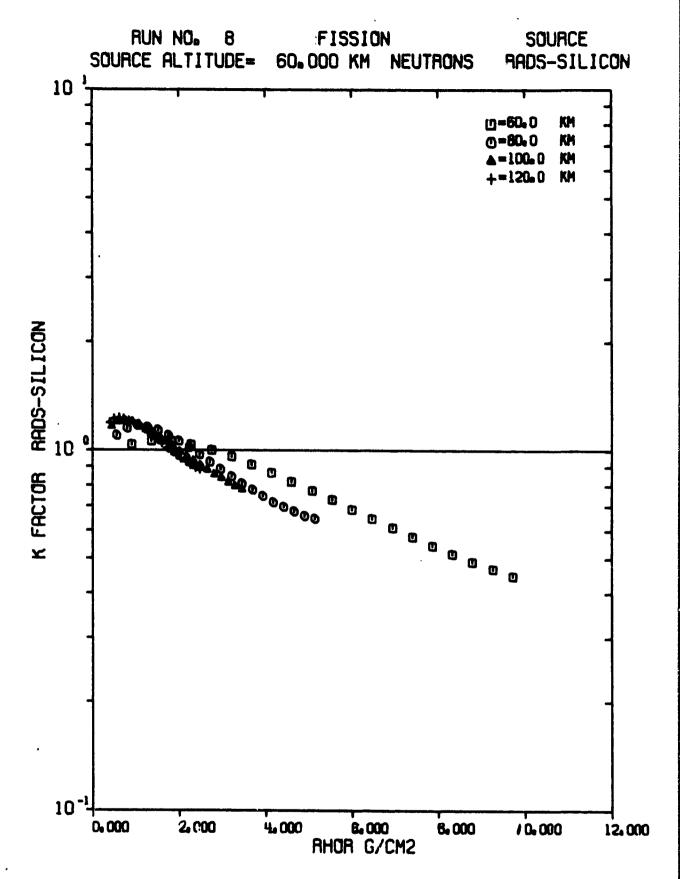


FIGURE U-34 MORSALK FIT DATA-NEUTRON SILICON K-FACTOR FISSION SOURCE IN REAL AIR AT 60.0 KM. SAMPLING ALTITUDES 60,80,100, AND 120 KM.

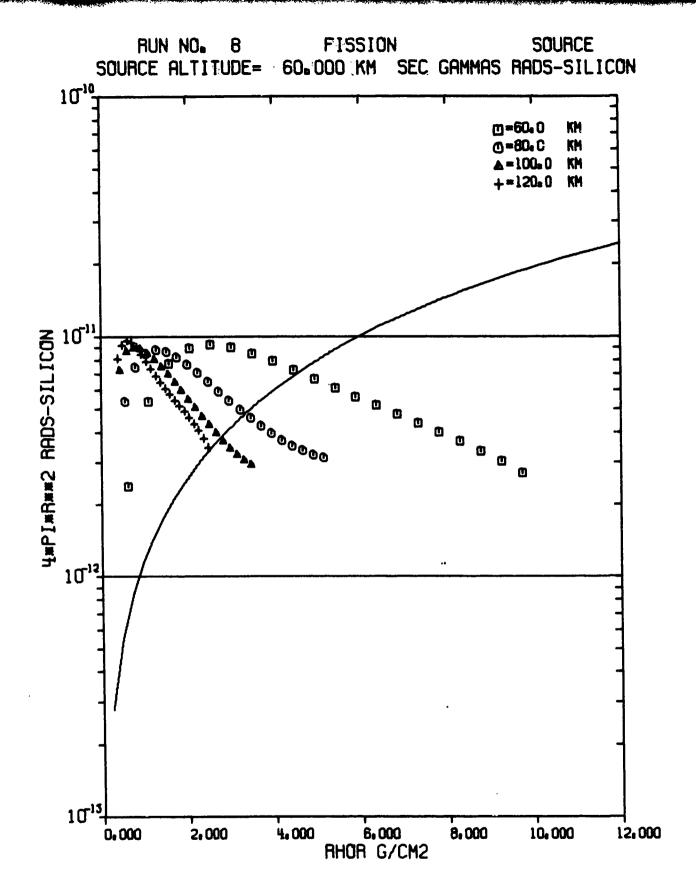


FIGURE 3-35 MORSAIR FIT DATA-4PIR**2 GAMMA SI DOSE, FISSION SOURCE IN REAL AIR AT 60.8 KM. SAMPLING ALTITUDES 50.88,100, AND 120 KM.

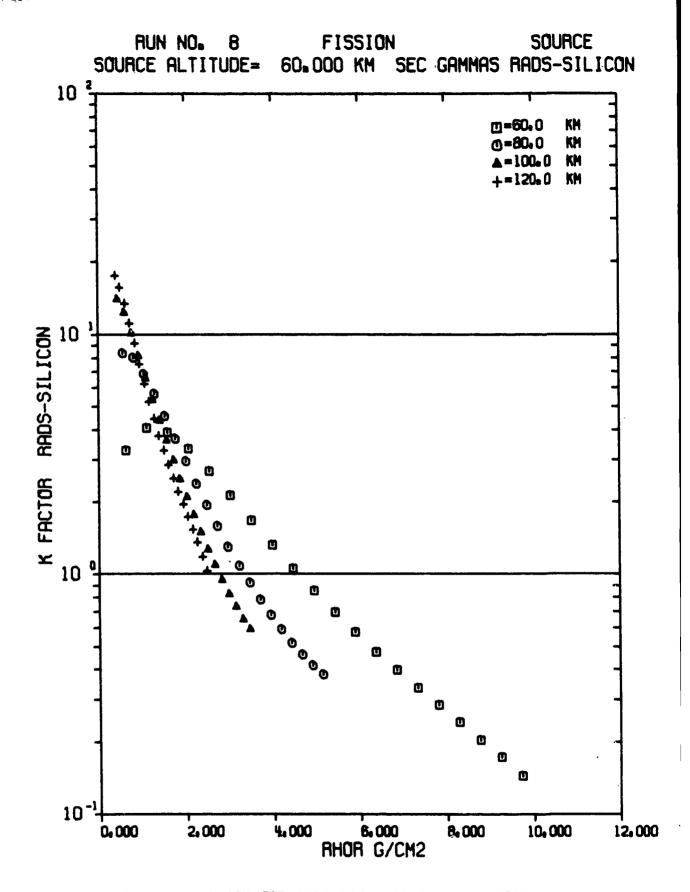


FIGURE C-36 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR, FISSION SOURCE IN REAL AIR AT 60.0 KM. SAMPLING ALTITUDES 60,80,100, AND 120 KM.

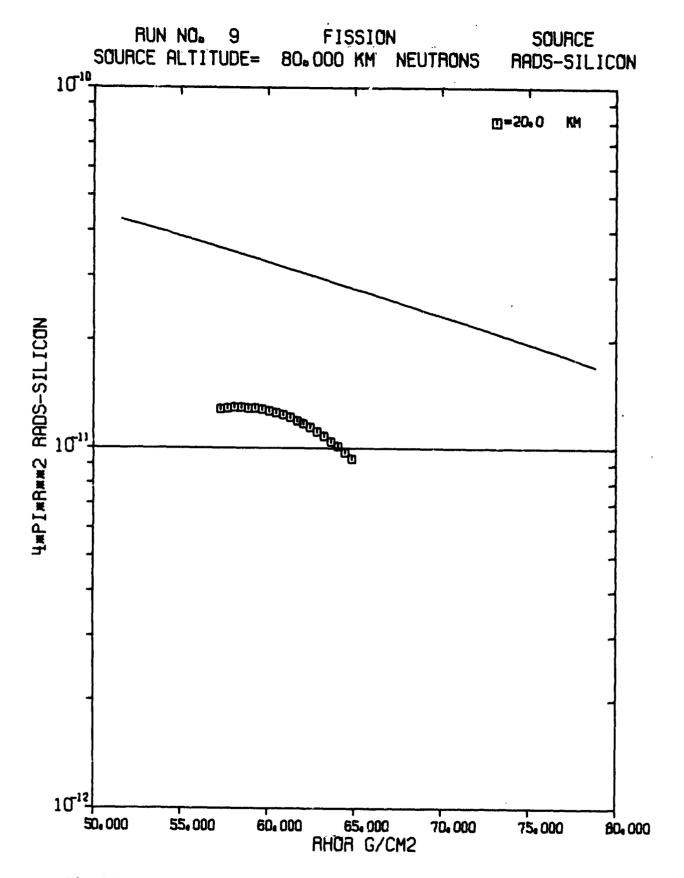


FIGURE C-37 MORSAIR FIT DATA-4PIR**2 NEUTRON SI DOSE. FISSION SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 20 KILOMETERS.

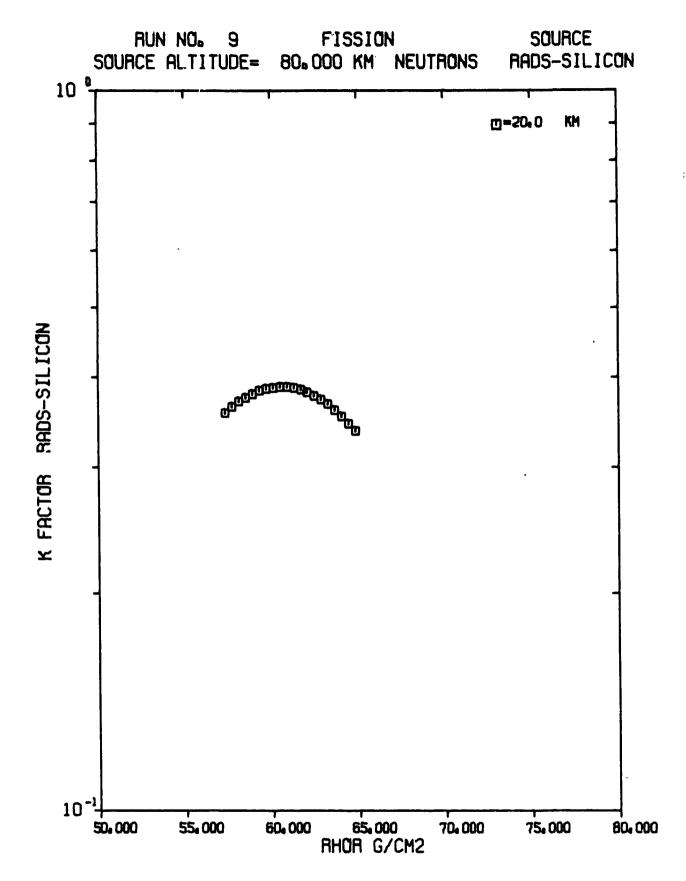


FIGURE C-38 MORSAIR FIT DATA-NEUTRON SILICON K-FACTOR FISSION SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 20 KILOMETERS.

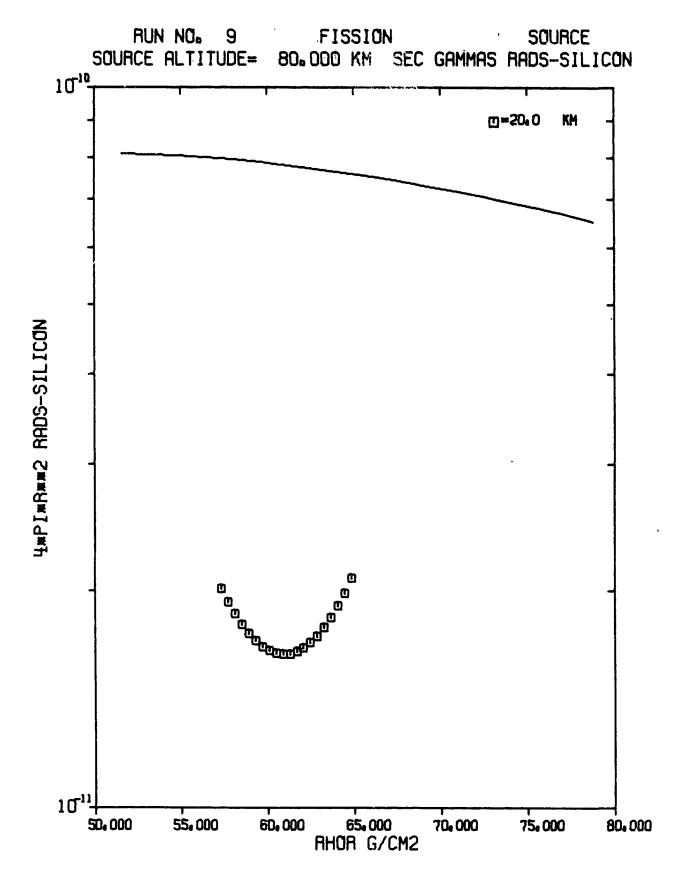


FIGURE C-39 MORSAIR FIT DATA-4PIR**2 GAMMA SI DOSE, F1SS10N SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 20 KILOMETERS.

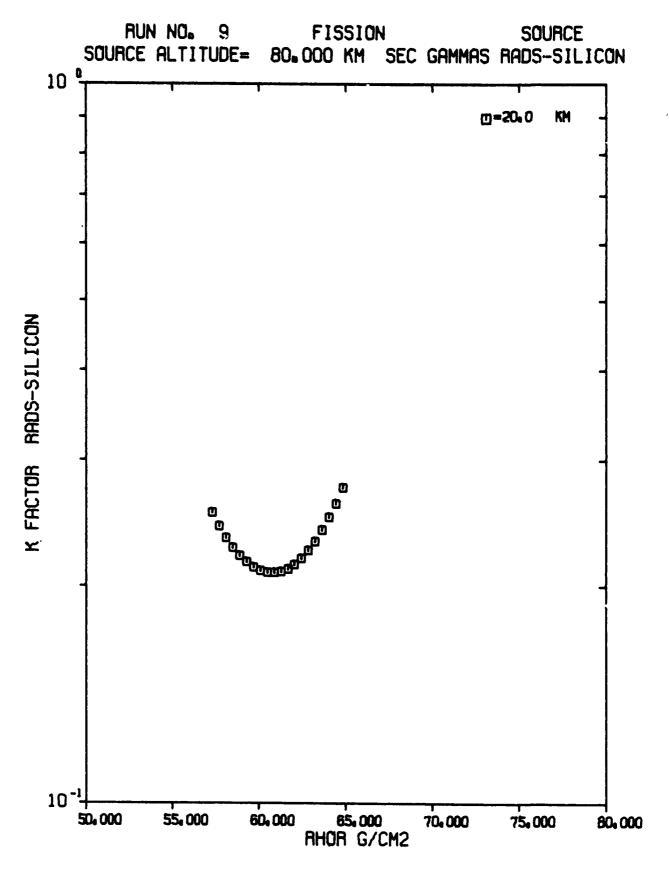


FIGURE 3-40 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR, FISSION SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 20 KILOMETERS.

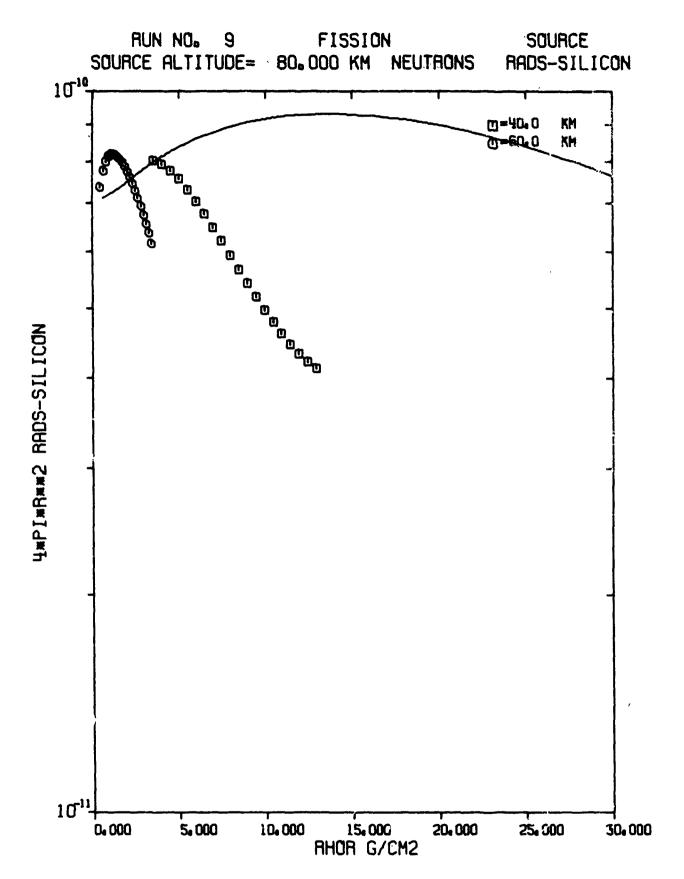


FIGURE C-41 MORSAIR FIT DATA-4PIR**2 NEUTRON SI DOSE, FISSION SOURCE IN RIAL AIR AT 80.0 KM. SAMPLING ALTITUDES 40 AND 60 KM.

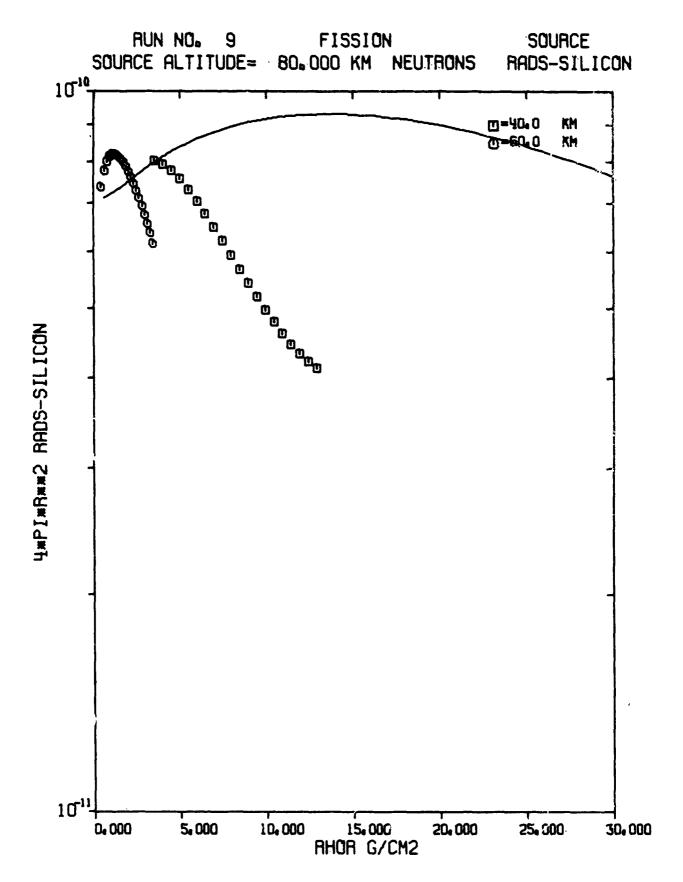


FIGURE C-41 MORSAIR FIT DATA-4PIR**2 NEUTRON SI DOSE, FISSION SOURCE IN RIAL AIR AT 80.0 < M. SAMPLING ALTITUDES 40 AND 60 km.

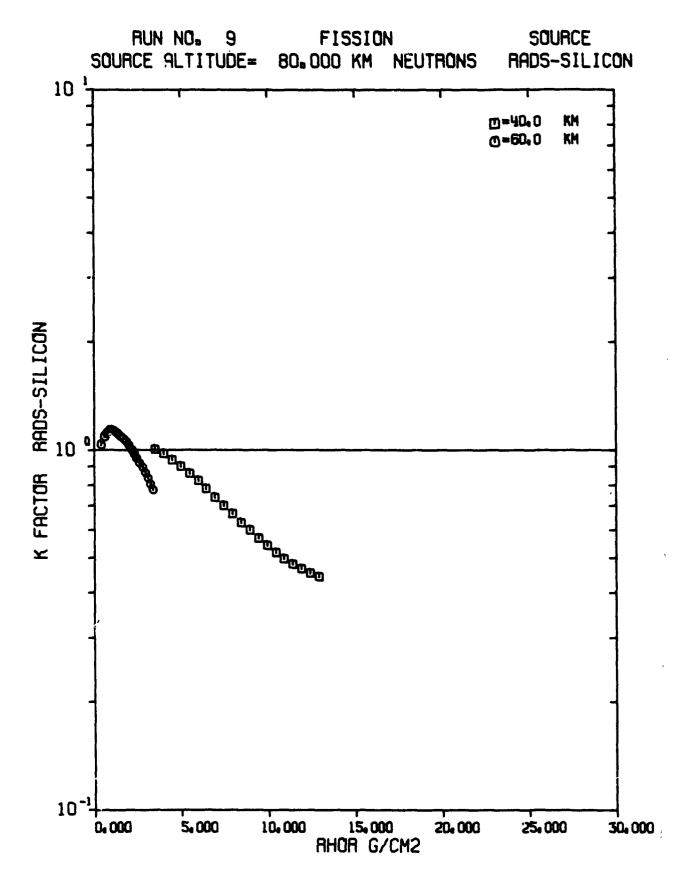


FIGURE C-42 MORSAIR FIT DATA-NEUTRON SILICON K-FACTOR FISSION SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 40 AND 60 KM.

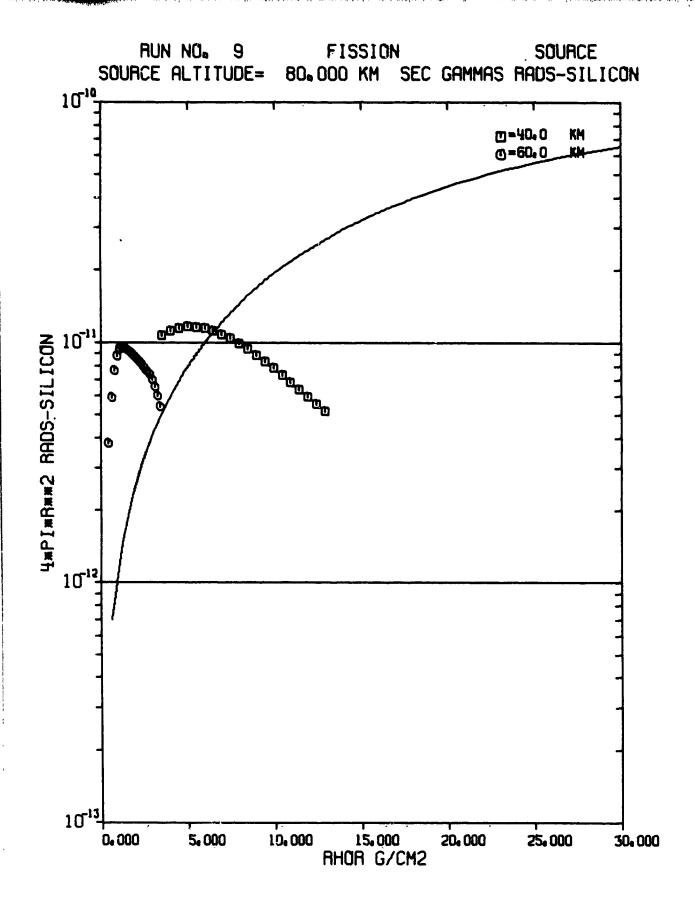


FIGURE 0-43 MORSALE FIT DATA-4PIR**2 GAMMA SI BOSE. FISSION SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 46 AND 60 KM.

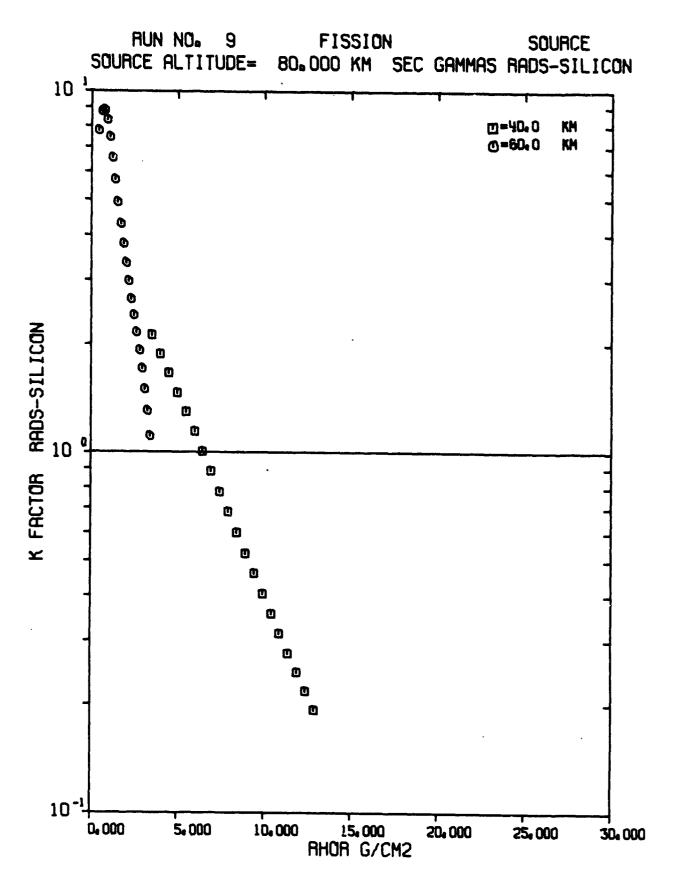


FIGURE C-44 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR, FISSION SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 40 AND 60 KM.

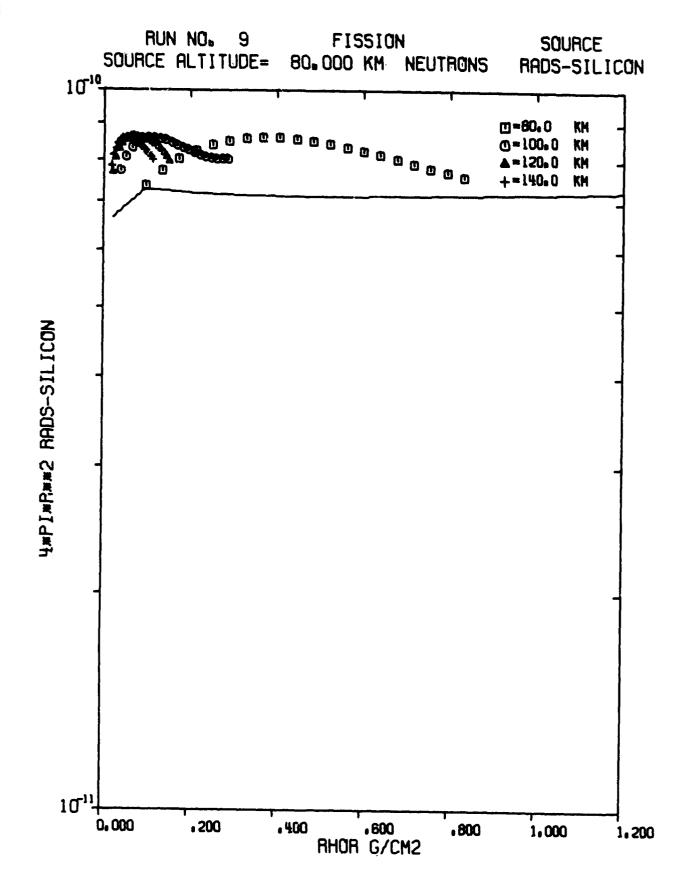


FIGURE U-45 MORSAIR FIT DATA-4PIR**2 NEUTRON SI DOSE, FISSION SOURCE IN R_AL AIR AT 80.0 KM. SAMPLING ALTITUDES 80.100,120, AND 143 KM.

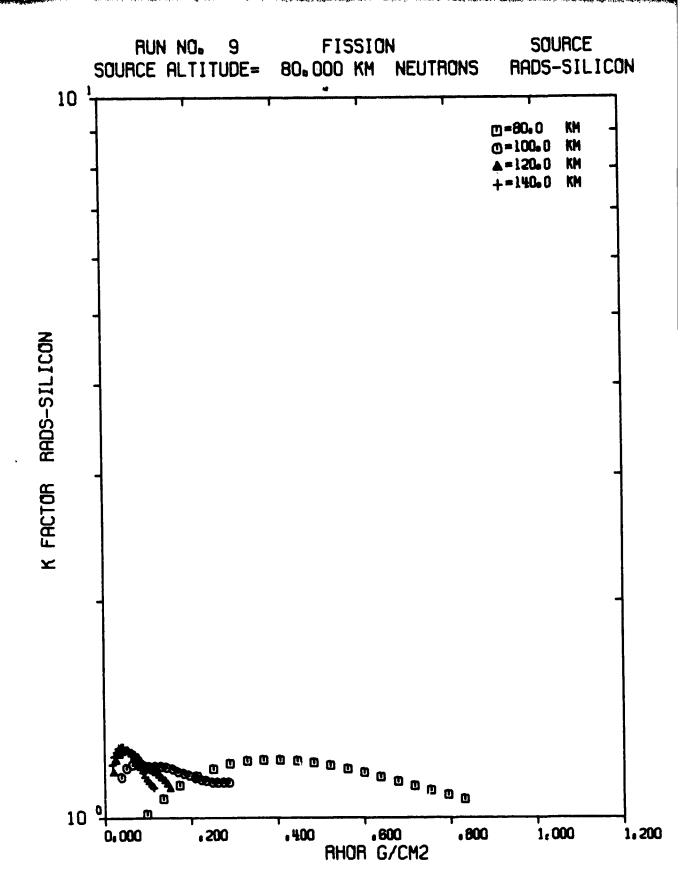


FIGURE U-46 MORSAIR FIT DATA-NEUTRON SILICON K-FACTOR FISSION SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 80.100.120. AND 140 KM.

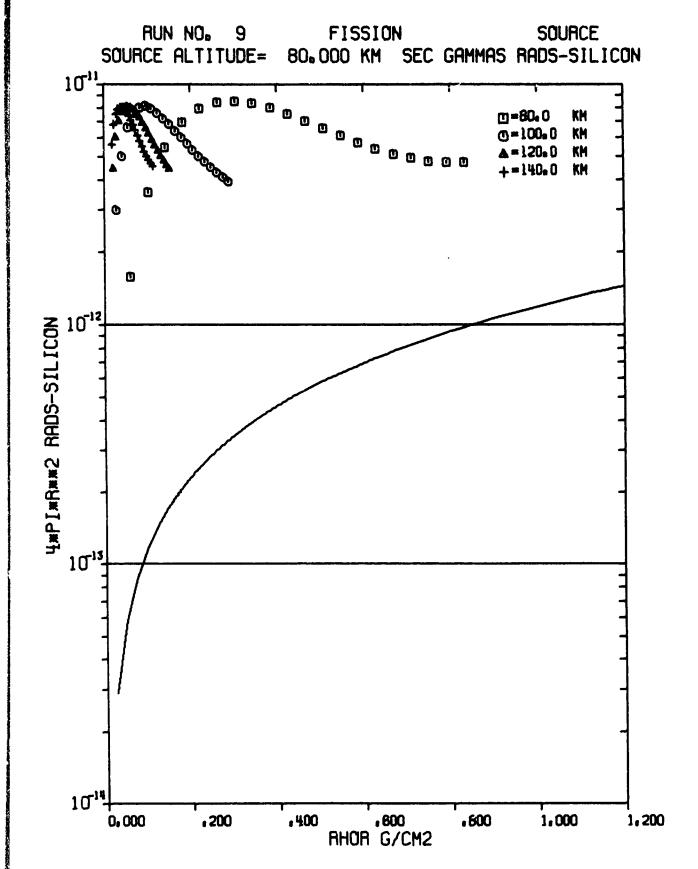


FIGURE C-47 MORSHIR FIT DATA-4PIR**2 GAMMA SI BOSE, FISSION SOURCE IN REAL AIR AT 80.3 KM. SAMPLING ALTITUDES 30,100,120, AND 145 KM.

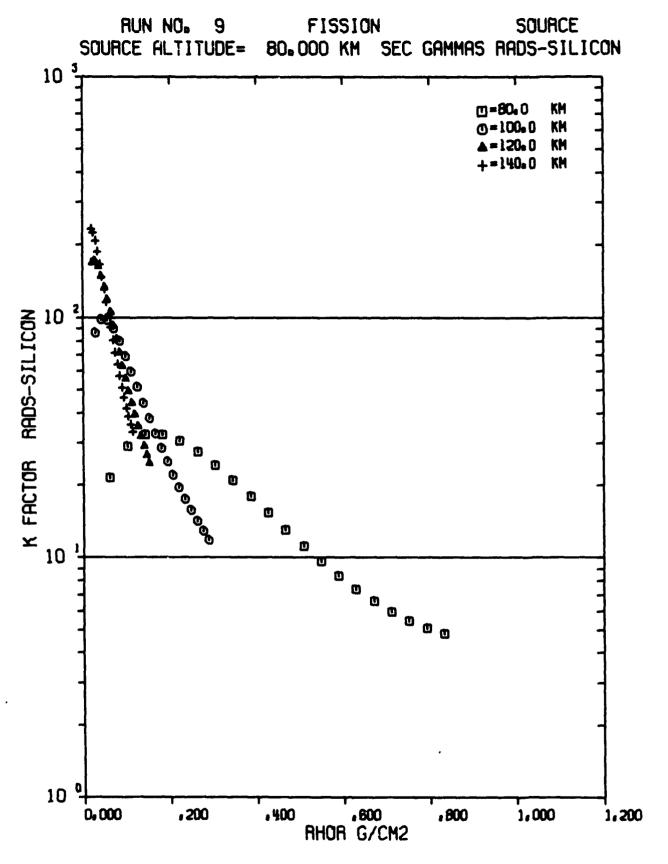


FIGURE C-48 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR, FISSION SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 80,100,120, AND 140 KM.

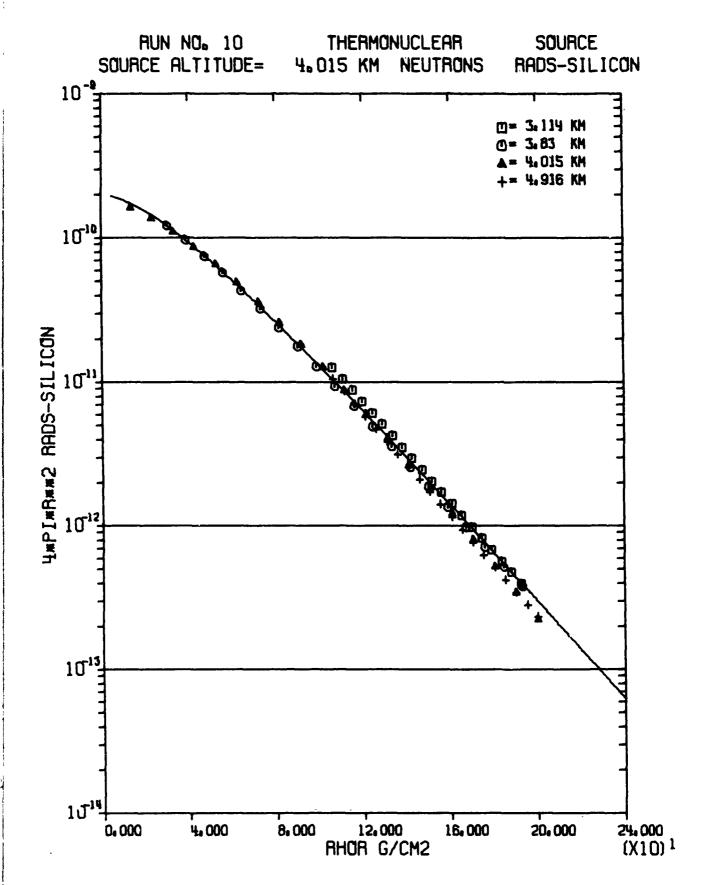


FIGURE C-49 MORSAIR FIT DATA-4PIR**2 NEUTRON SI DOSE. THERMONUCL SOURCE IN HOMO AIR AT 4.0 KM. ALL SAMPLING ALTITUDES

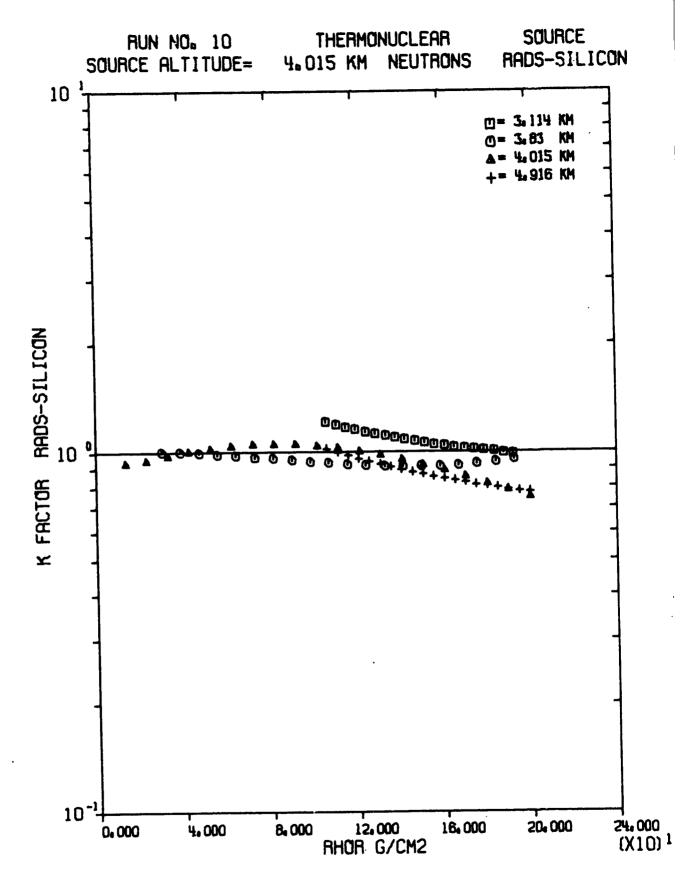
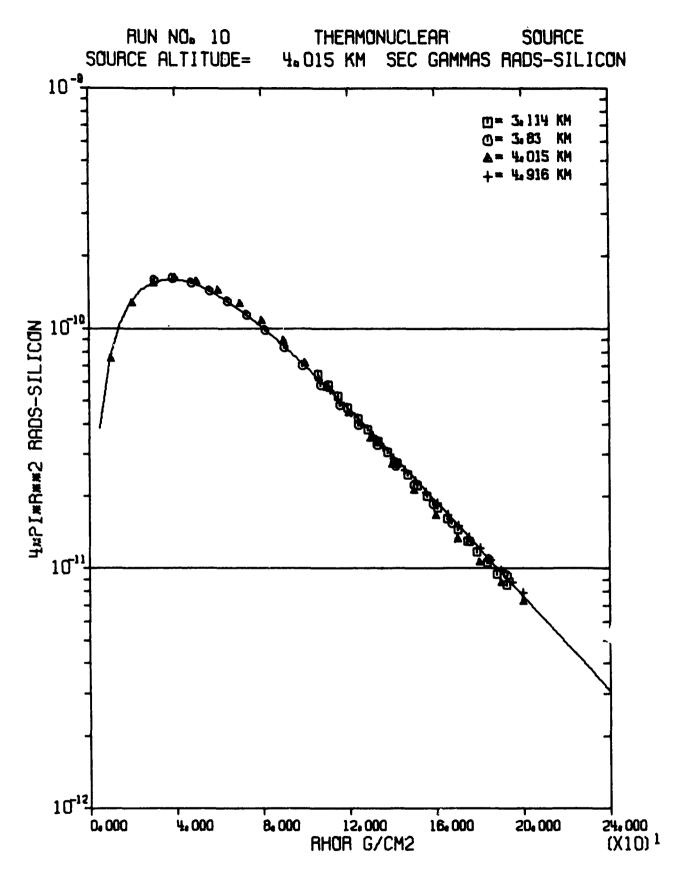


FIGURE C-50 MORSAIN FIT DATA-NEUTRON SILICON K-FACTOR THERMOMUCE SOURCE IN HOMO AIR AT 4.0 KM. ALL SAMPLING ALTITUDES



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FIGURE C-51 MORSAIK FIT DATA-4PIR**2 GAMMA SI DOSE, THERMONUCE SOURCE IN HOMO AIR AT 4.0 KM. ALL SAMPLING ALTITUDES

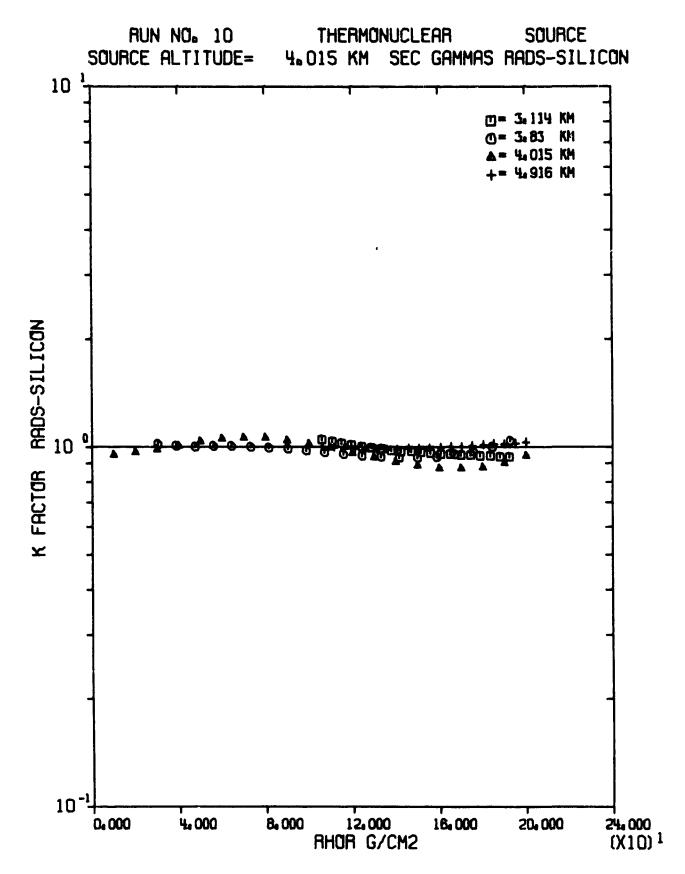


FIGURE C-52 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR, THERMONUCL SOURCE IN HOMO AIR AT 4.0 KM. ALL SAMPLING ALTITUDES

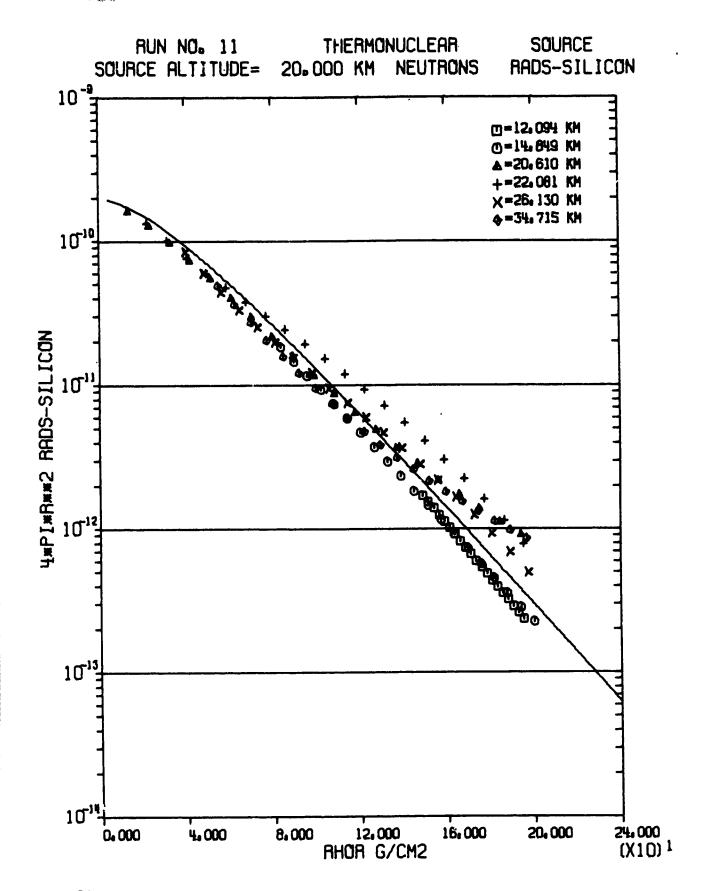


FIGURE 3-93 MORSAIR FIT DATA-4PIR**2 NEUTRON SI DOSE. THERMONUCL SOURCE IN REAL AIR AT 20.0 KM. ALL GAMPLING ALTITUDES

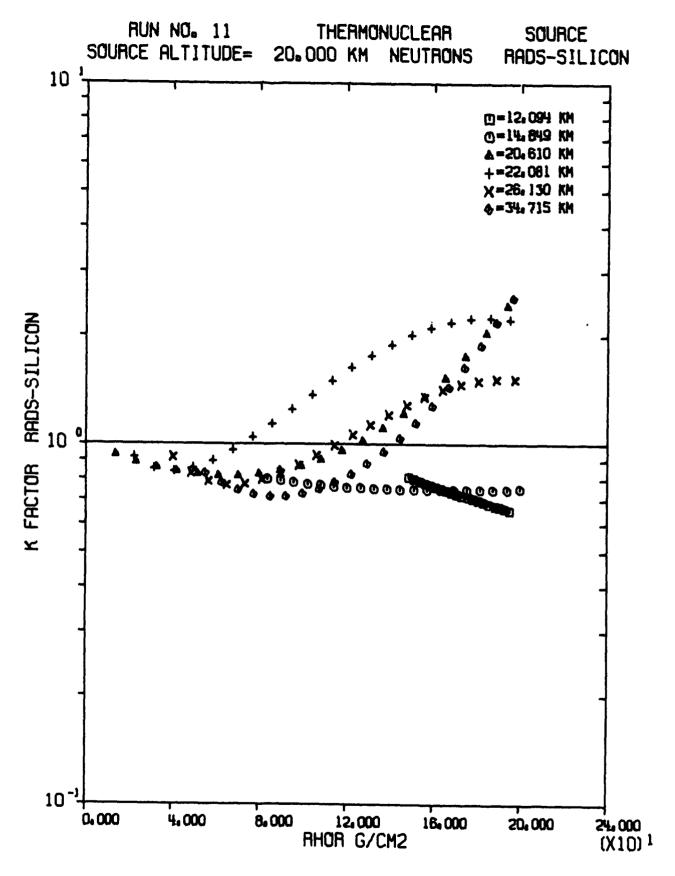


FIGURE C->+ MURSAIR FIT DATA-NEUTRON SILICON K-FACTOR THÉRMONUCL SOURCE IN REAL AIR AT 20.0 KM. ALL SAMPLING ALTITUDES

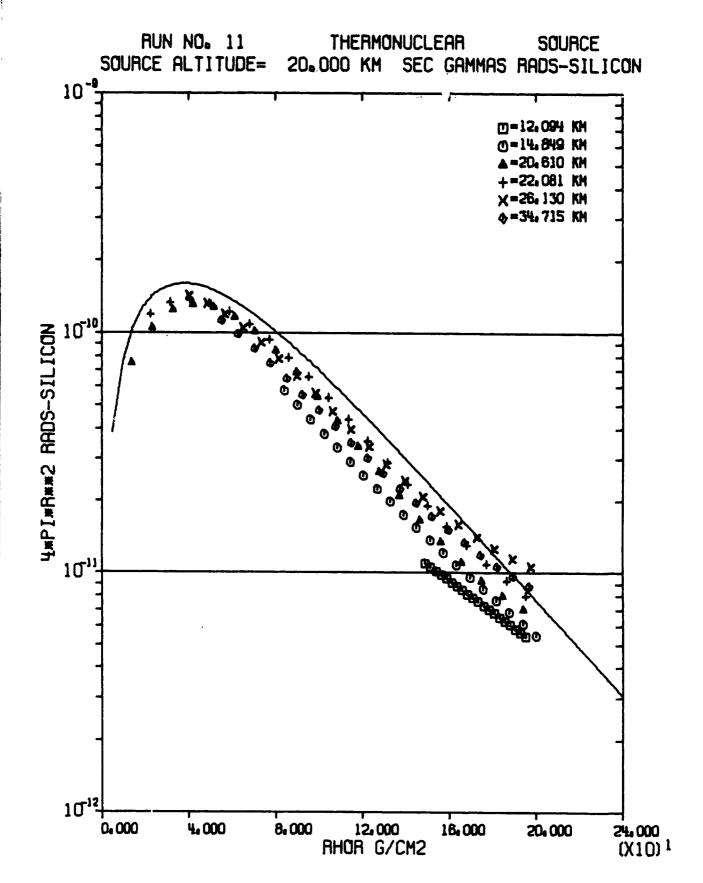


FIGURE C-55 MURSAIR FIT DATA+4PIR**2 GAMMA SI DOSE, THERMONUCL SOURCE IN REAL AIR AT 20.0 KM. ALL SAMPLING ALTITUDES

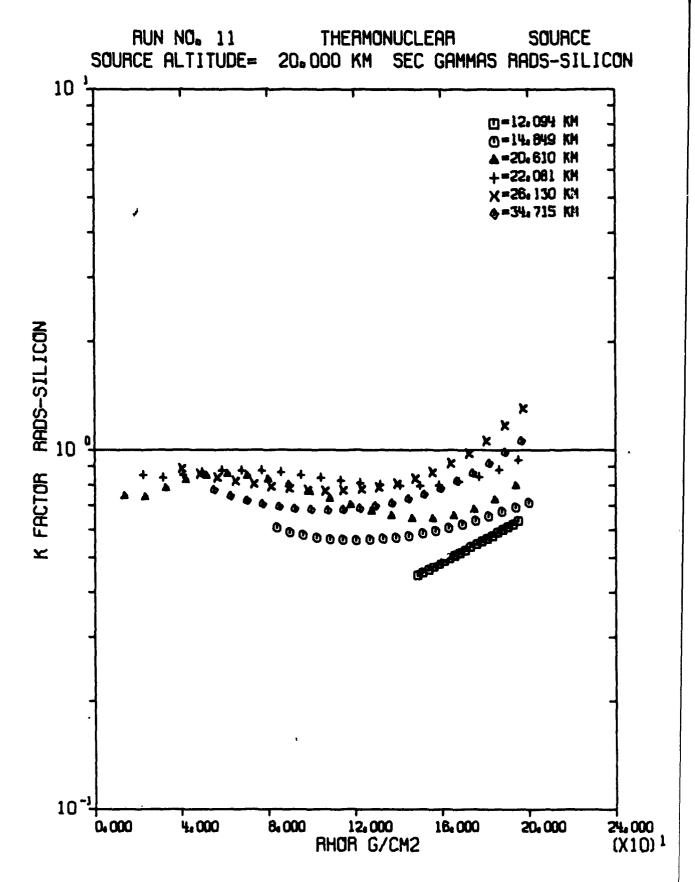


FIGURE C-56 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR.
THERMONUCL SOURCE IN REAL AIR AT 20.0 KM.
ALL JAMPLING ALTITUDES

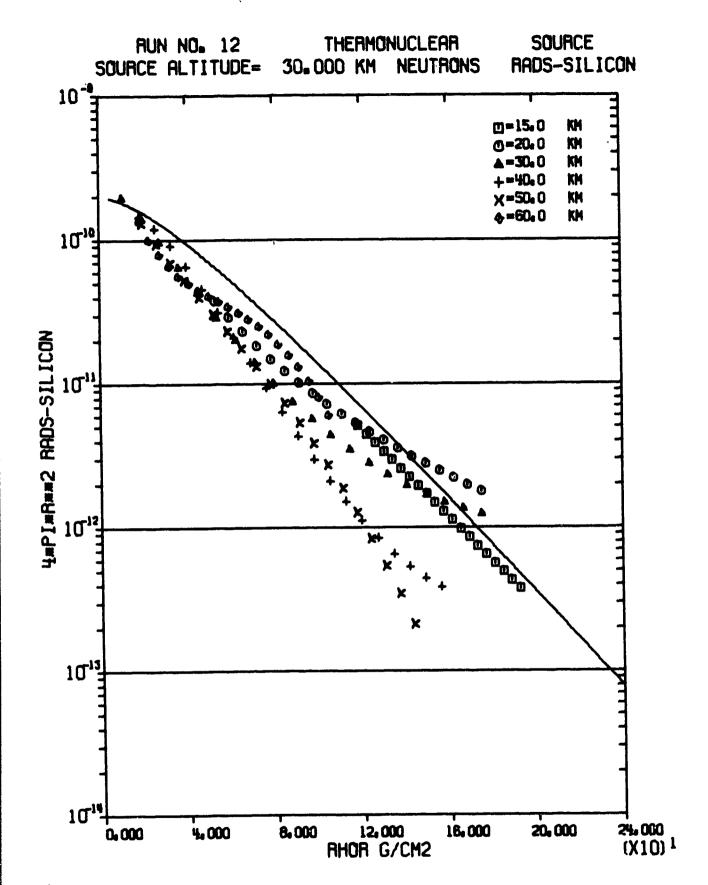


FIGURE 3-57 MORSAIR FIT DATA-4PIR**2 NEUTRON SI DOSE, THERMONUCL SOURCE IN REAL AIR AT 30.0 KM. ALL SAMPLING ALTITUDES

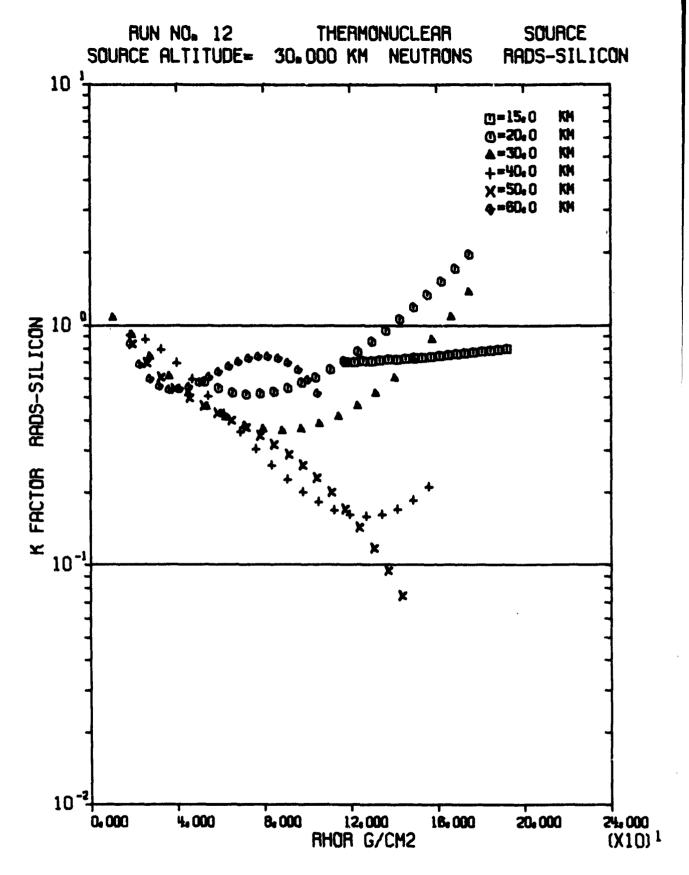


FIGURE C-58 40KSAIR FIT DATA-NEUTRON SILICON K-FACTOR THERMONUCL SOURCE IN REAL AIR AT 30.0 KM. ALL SAMPLING ALTITUDES

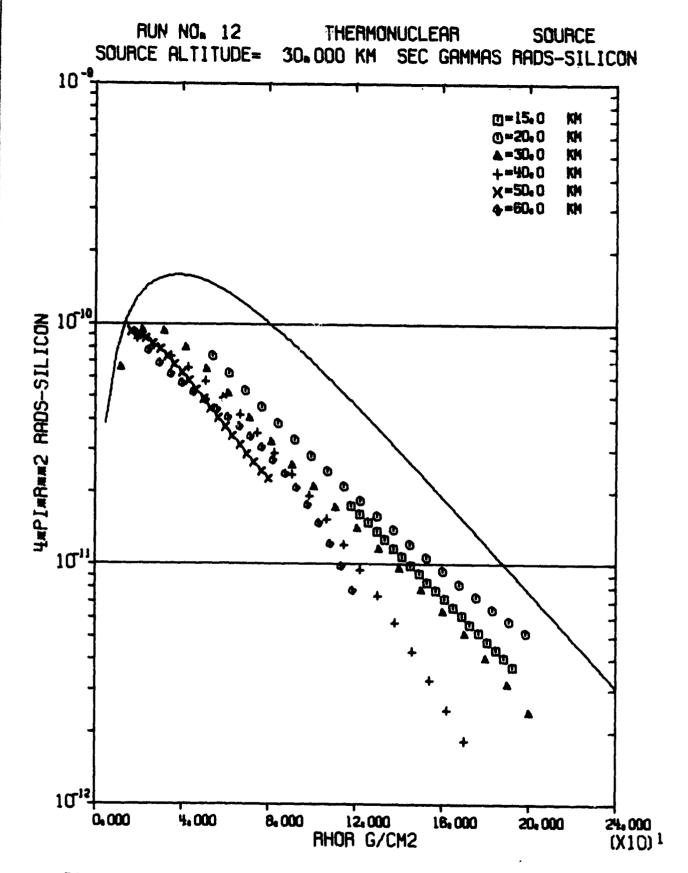
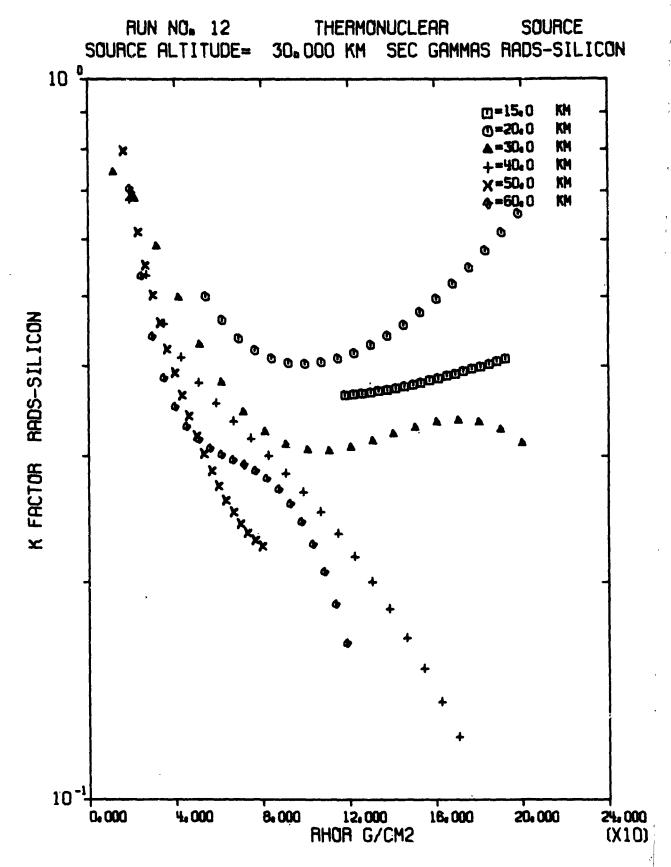


FIGURE 6-59 MORSAIR FIT DATA-4PIR**2 GAMMA SI DOSE,
TO ONUCL SOURCE IN REAL AIR AT 30.0 KM.
4LL SAMPLING ALTITUDES



FIGU 1-06 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR, THERMONUCL SOURCE IN REAL AIR AT 30.0 KM. ALL SAMPLING ALTITUDES

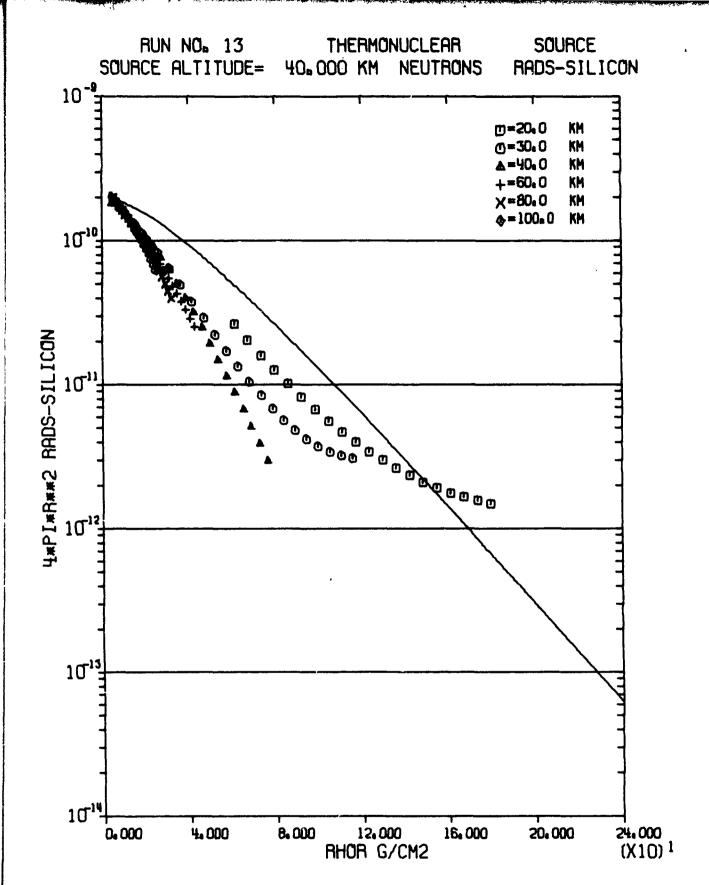


FIGURE G-61 MORSAIR FIT DATA-4PIR**2 NEUTRON SI DOSE, THERMONUCL SOURCE IN REAL AIR AT 46.0 KM. ALL SAPPLING ALTITUDES

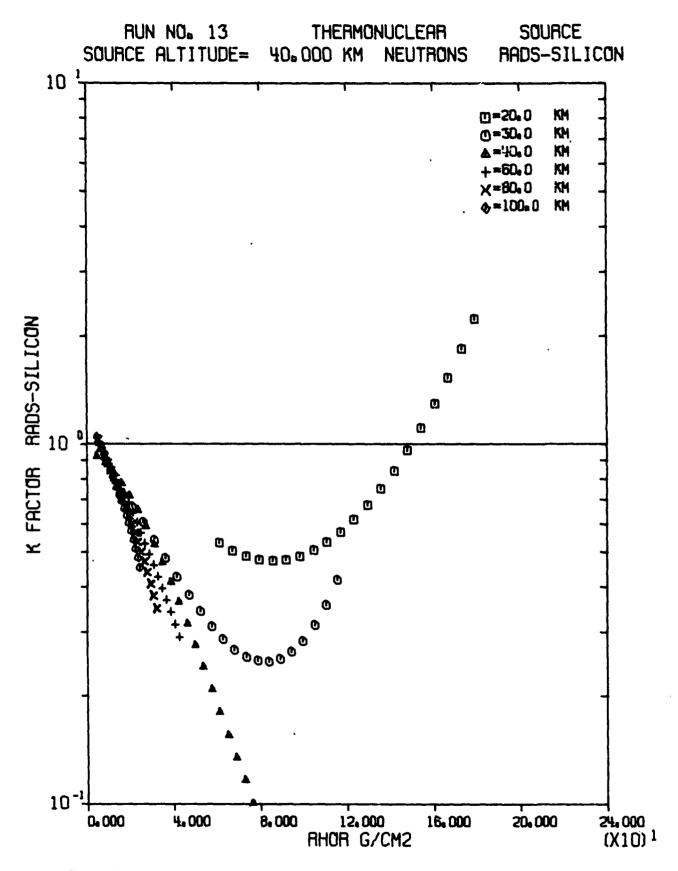


FIGURE 3-62 MURSAIR FIT DATA-NEUTRON SILICON K-FACTOR THERMUNUCL SOURCE IN REAL AIR AT 40.0 KM. ALL SAMPLING ALTITUDES

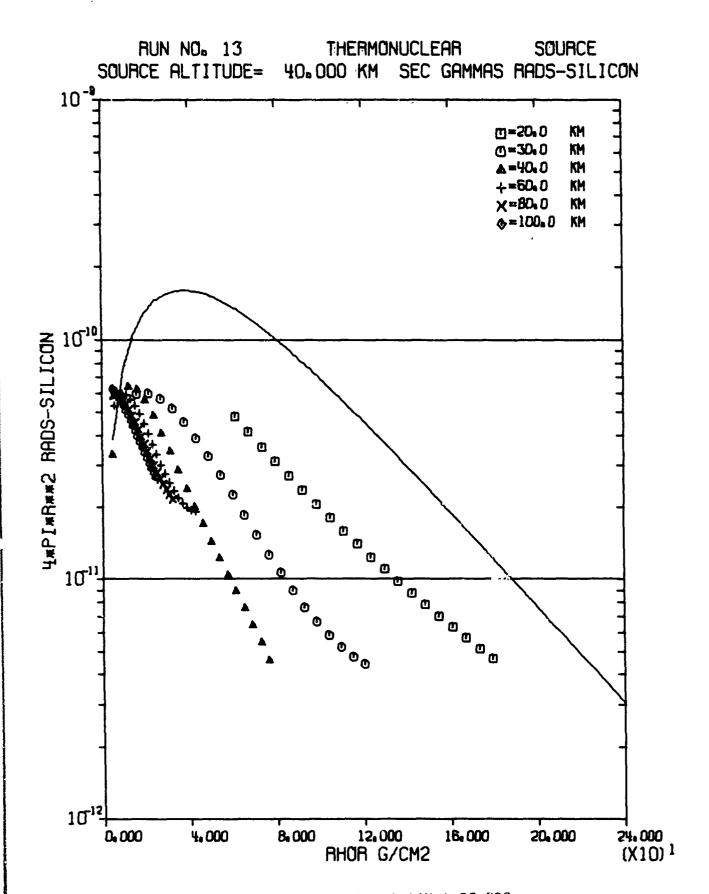


FIGURE C-63 MORSAIR FIT DATA-4PIR**2 GAMMA SI DOSE, THERMONUCL SOURCE IN REAL AIR AT 40.0 KM. ALL SAMPLING ALTITUDES

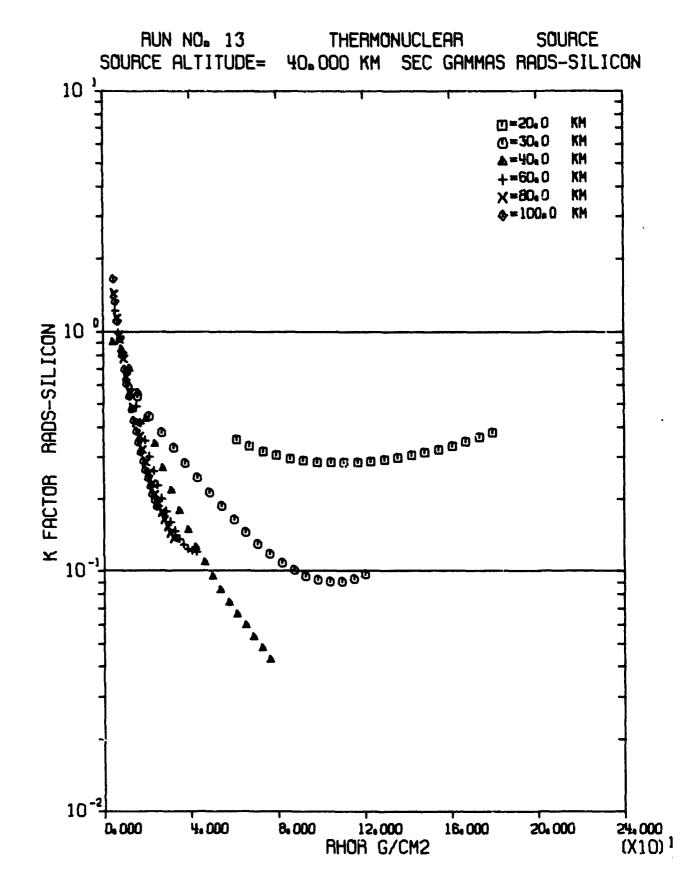


FIGURE C-64 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR, THÉRMONUCL SOURCE IN REAL AIR AT 40.0 KM. ALL SAMPLING ALTITUDES

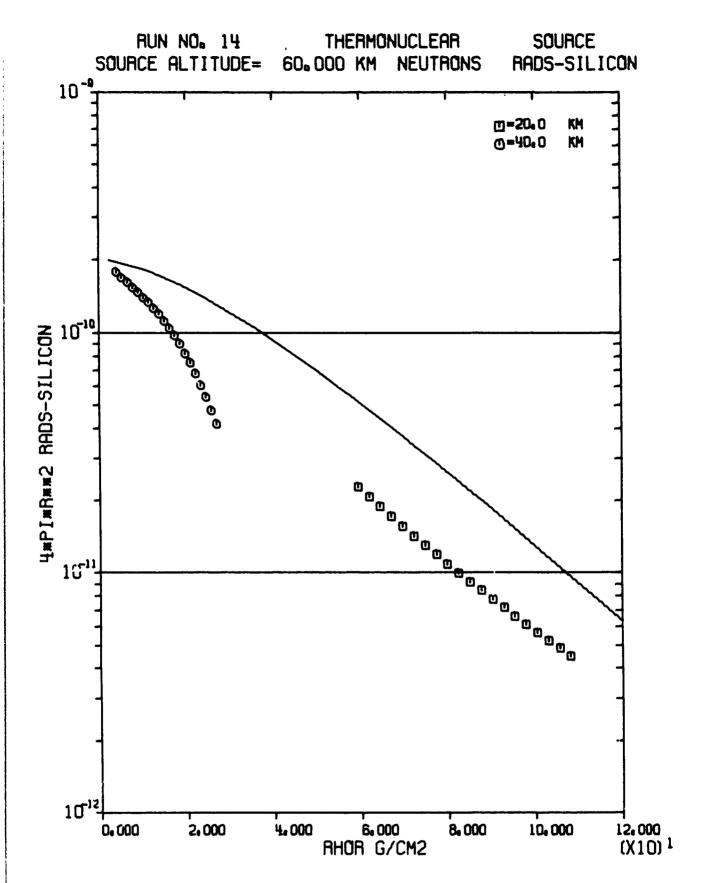


FIGURE C-65 MORSAIR FIT DATA-4PIR**2 NEUTRON SI DOJE, THERMONUCL SOURCE IN REAL AIR AT 60.0 KM. SAMPLING ALTITUDES 20 AND 40 KM.

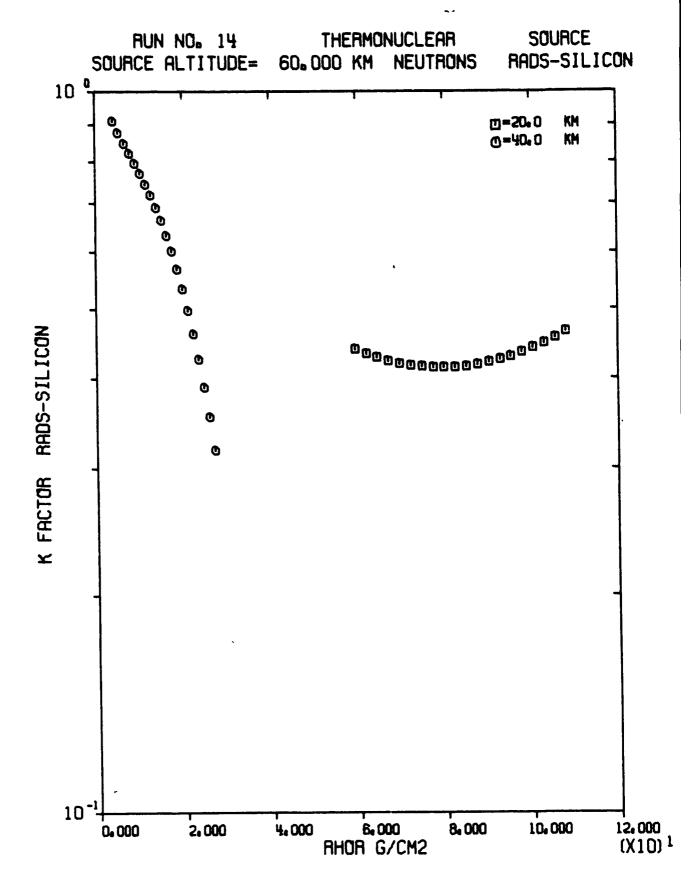


FIGURE C-66 MORSAIR FIT DATA-NEUTRON SILICON K-FACTOR THERMONUCL SOURCE IN REAL AIR AT 60.0 KM. SAMPLING ALTITUDES 20 AND 40 KM.

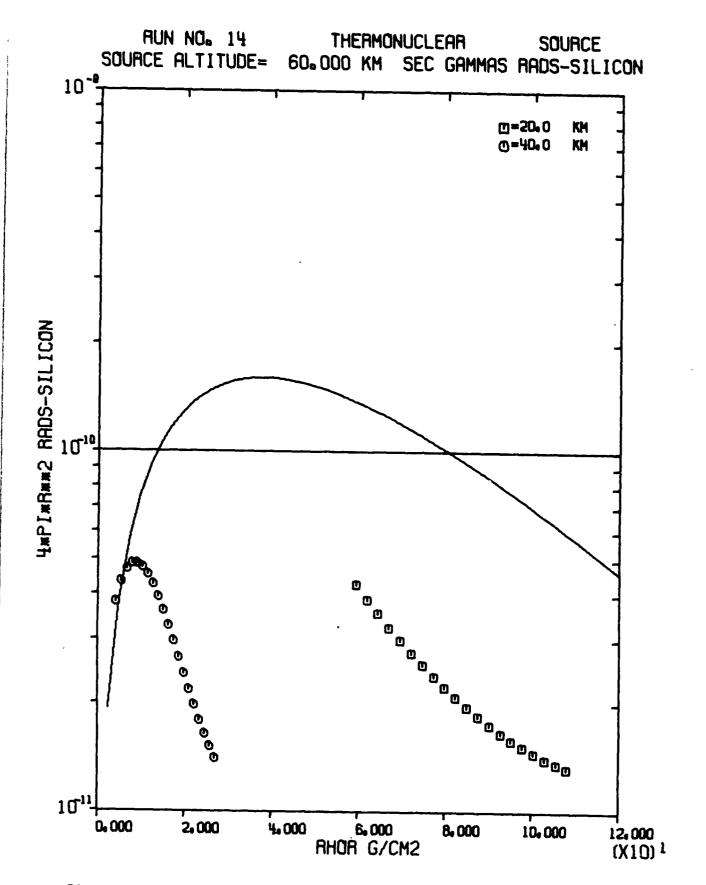


FIGURE 0-07 MORSAIR FIT DATA-4PIR**2 GAMMA SI DOSE. THERMONUCL SCURCE IN REAL AIR AT 60.0 KM. SAMPLING ALTITUDES 20 AND 40 KM.

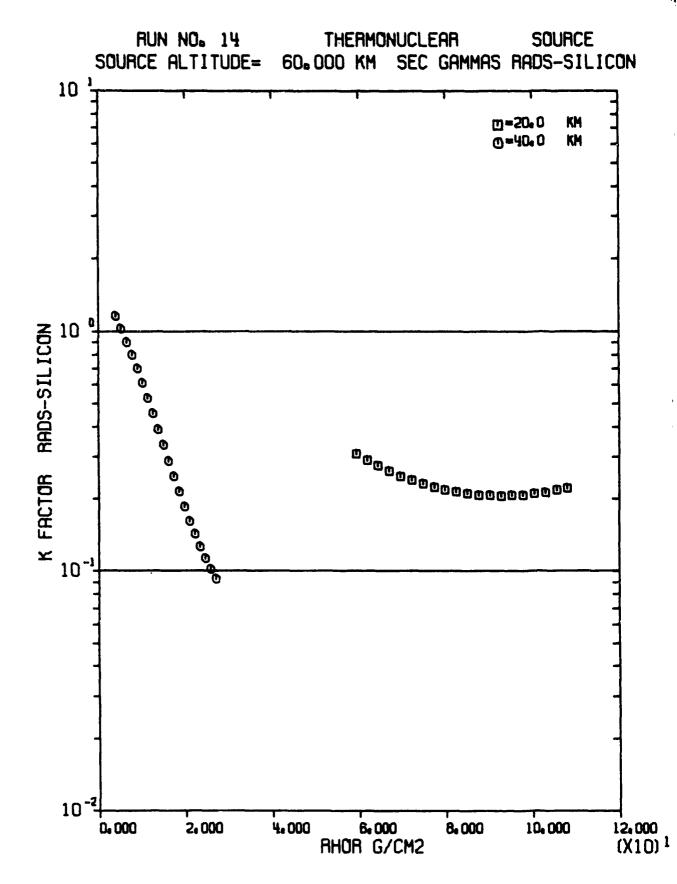


FIGURE C-68 MORSAIR FIT DA1A-GAMMA SILICON K-FACTOR.
THERMONUCL SOURCE IN REAL AIR AT 60.0 KM.
SAMPLING ALTITUDES 20 AND 40 KM.

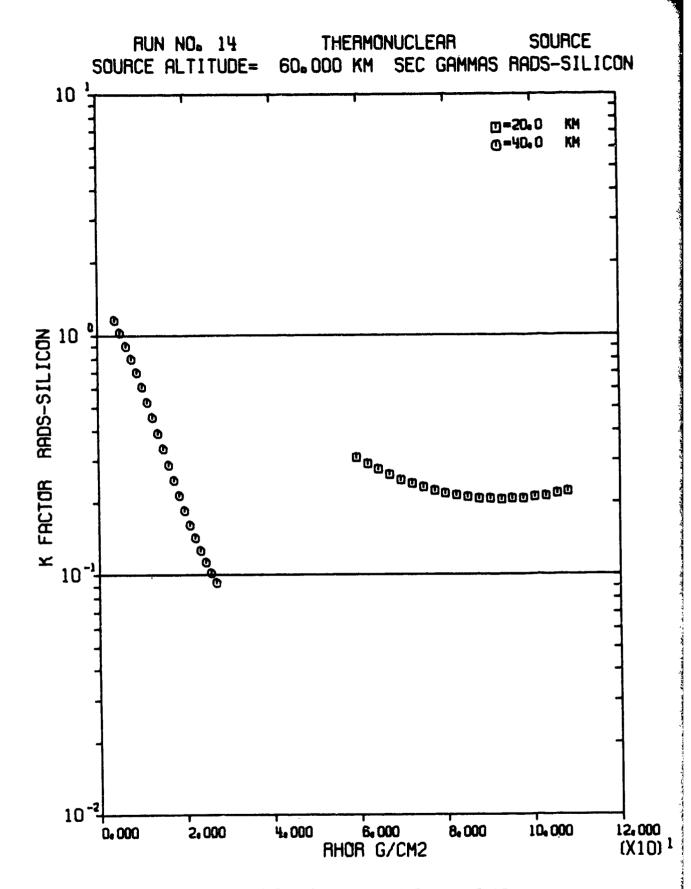


FIGURE C-68 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR, THERMONUCL SOURCE IN REAL AIR AT 60.0 KM. SAMPLING ALTITUDLS 20 AND 40 KM.

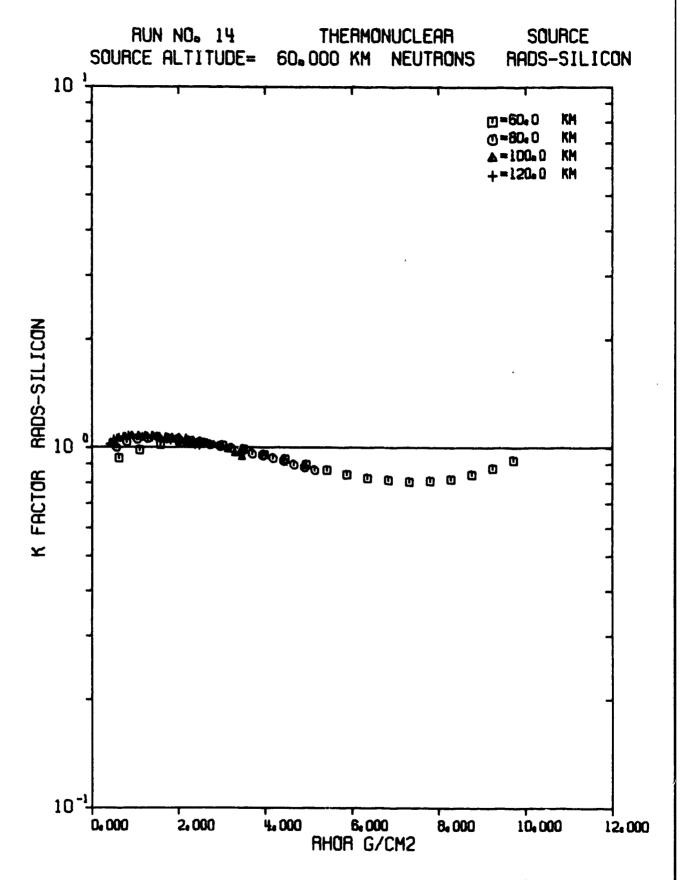


FIGURE C-70 MORSAIR FIT DATA-NEUTRON SILICON K-FACTOR THERMONUCL SOURCE IN REAL AIR AT 60.0 KM. SAMPLING ALTITUDES 50,80,100, AND 120 KM.

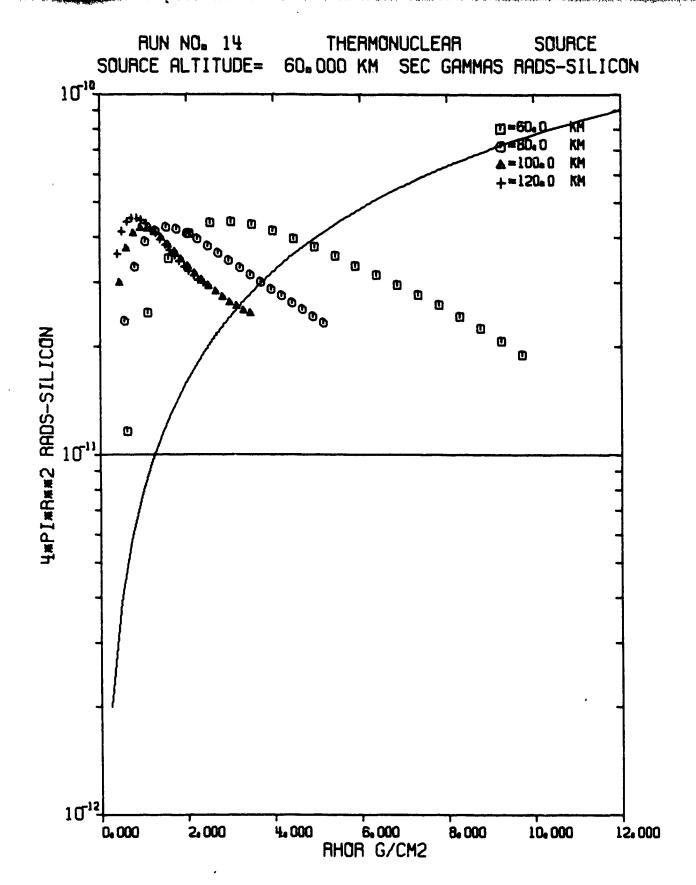


FIGURE C-71 MORSAIR FIT DATA-4PIR**2 GAMMA SI BOSE, THERMONUCL SOURCE IN REAL AIR AT 60.0 KM. SAMPLING ALTITUDES 50,80,100, AND 120 KM.

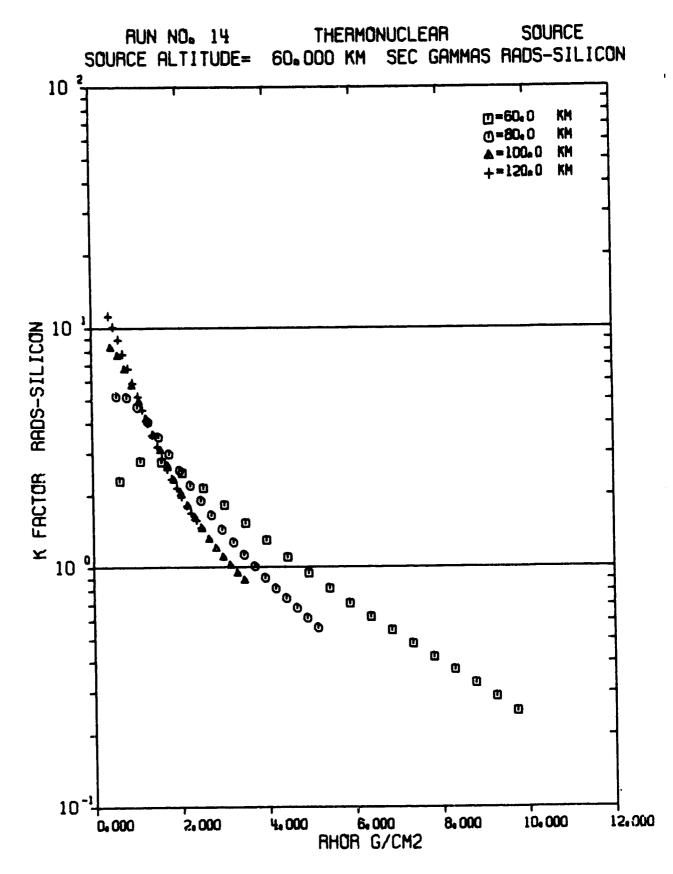


FIGURE C-72 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR, THERMONUCL SOURCE IN REAL AIR AT 60.0 KM. SAMPLING ALTITUDES 60,80,100, AND 120 KM.

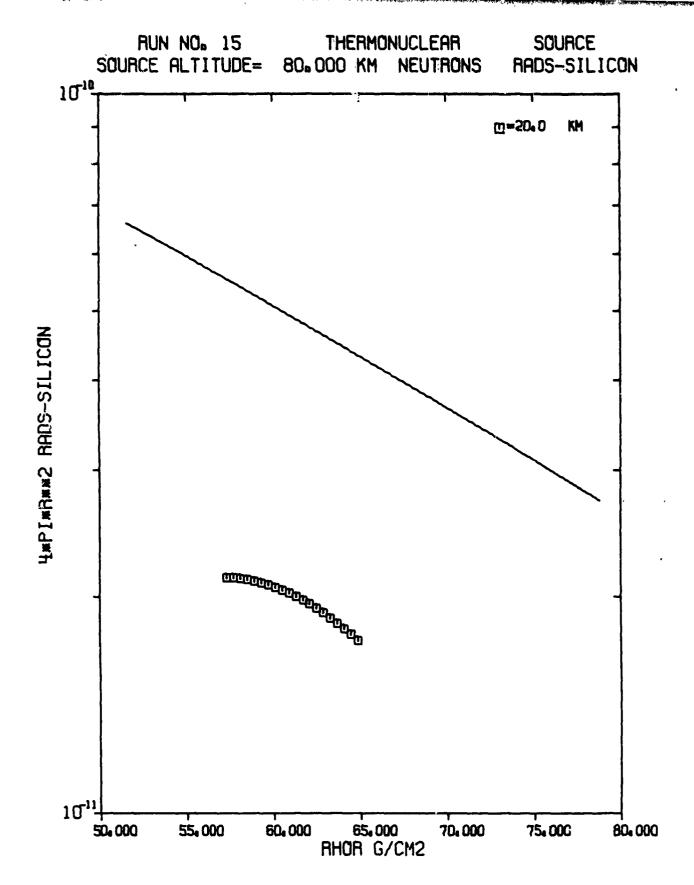


FIGURE 3-73 MORSAIR FIT DATA-4PIR**2 NEUTRON SI DOSE, THERMONUCL SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 20 KILOMETERS.

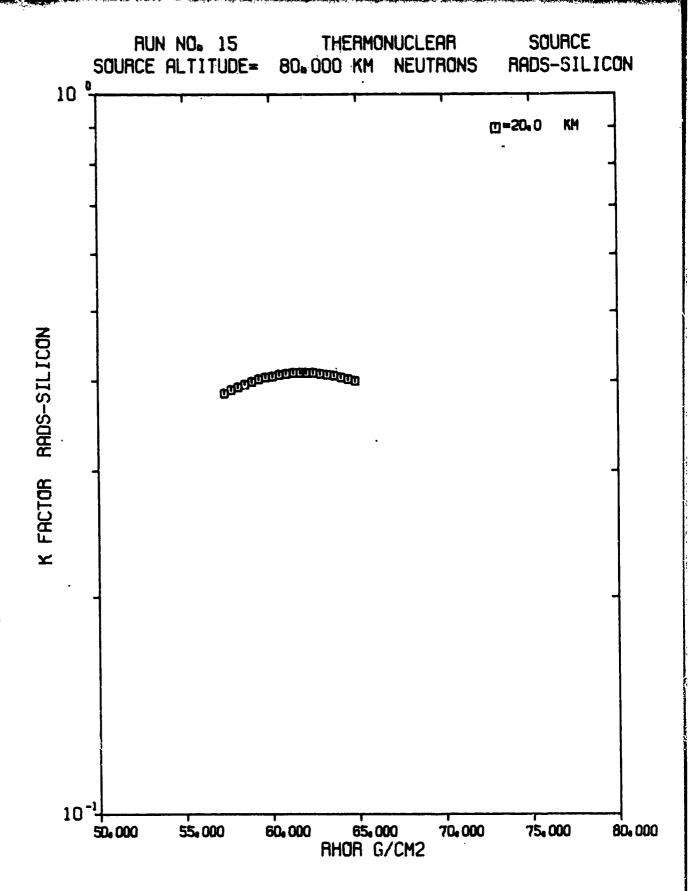


FIGURE C-74 MORSAIR FIT DATA-NEUTRON SILICON K-FACTOR THERMONUCL SOURCE IN REAL AIR AT 80.0 KM. JAMPLING ALTITUDES 20 KILOMETERS.

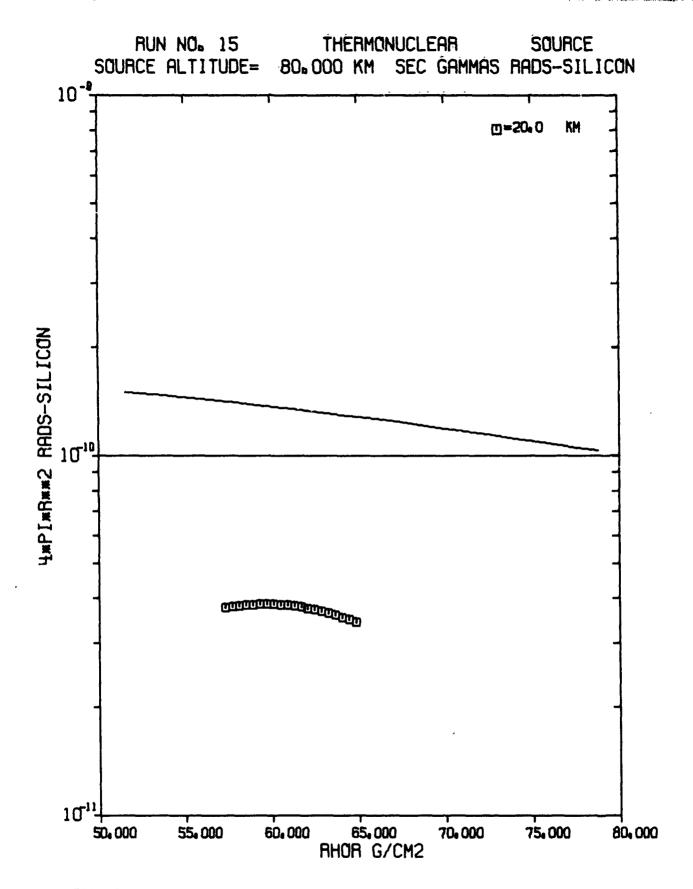


FIGURE C-75 MORSAIR FIT DATA-4PIR**2 GAMMA SI DOSE, THERMONUCL SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 20 KILOMETERS.

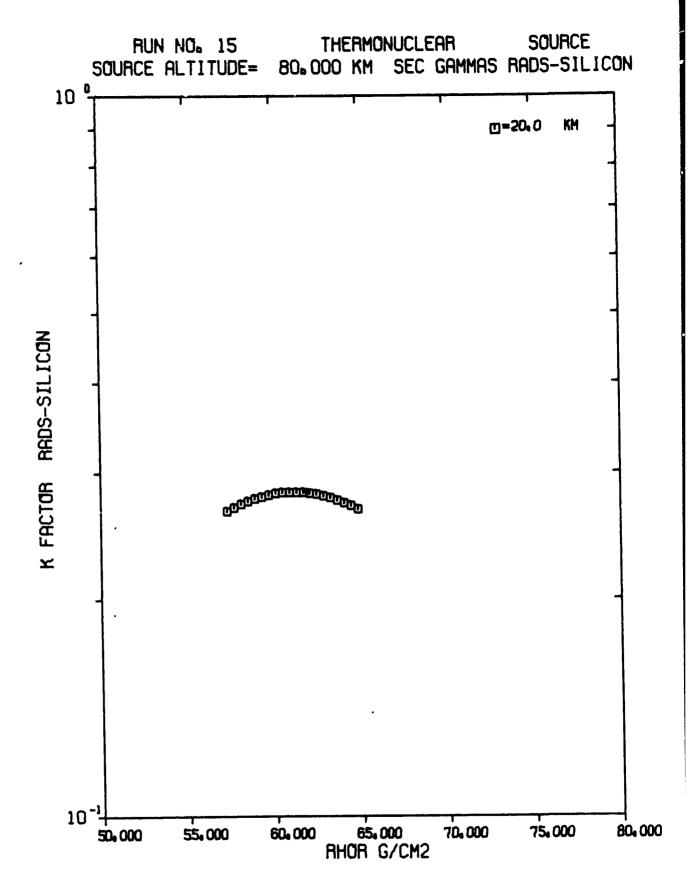


FIGURE C-76 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR, THERMONUCL SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 20 KILOMETERS.

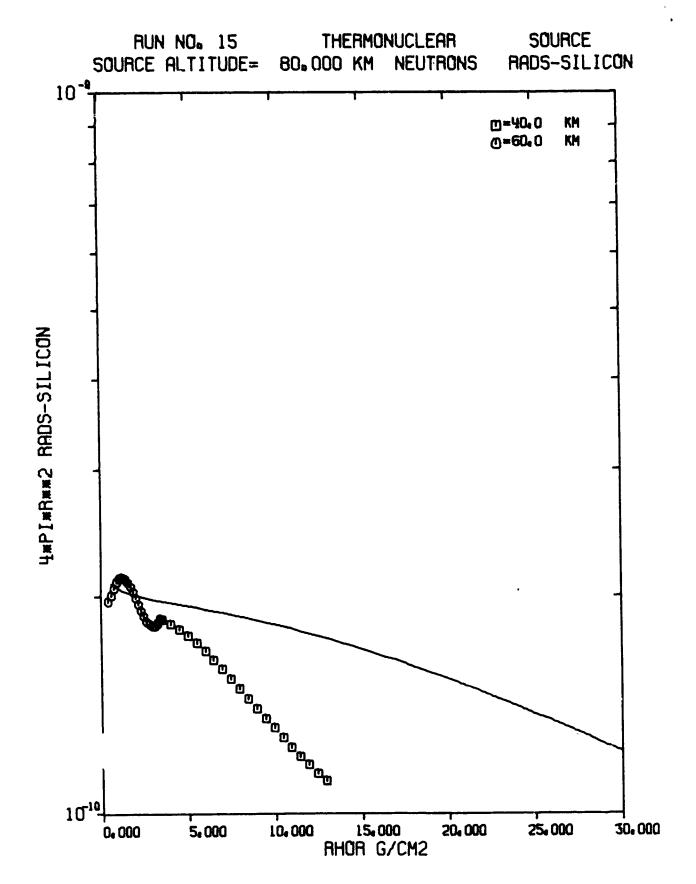


FIGURE C-77 MORSAIR FIT DATA-4PIR**2 NEUTRON SI DOSE. THERMONUCL SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 40 AND 60 kM.

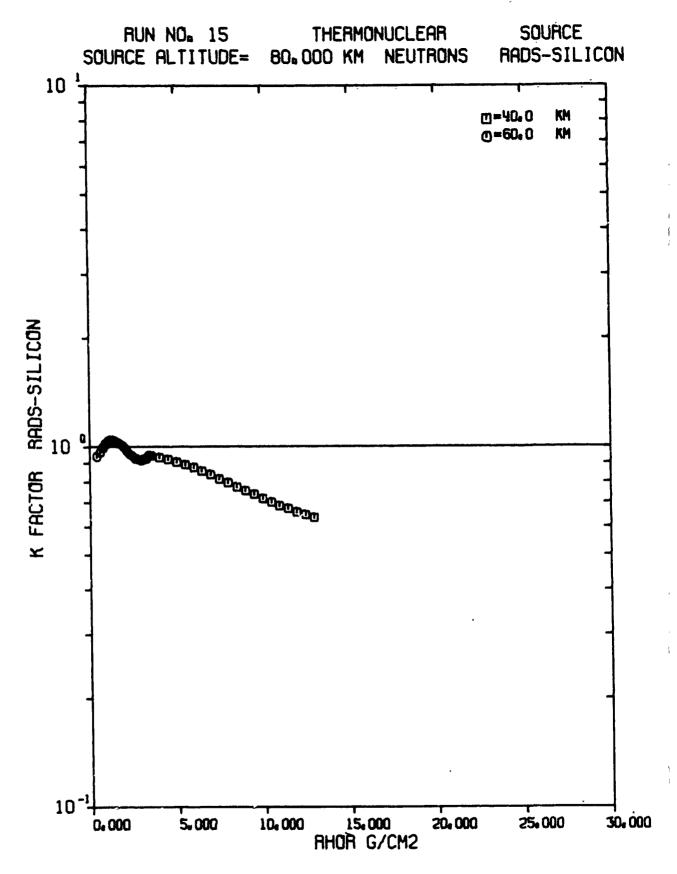


FIGURE 0-78 HORSAIR FIT DATA-NEUTRON SILICON K-FACTOR THERMONUCL SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 40 AND 60 KM.

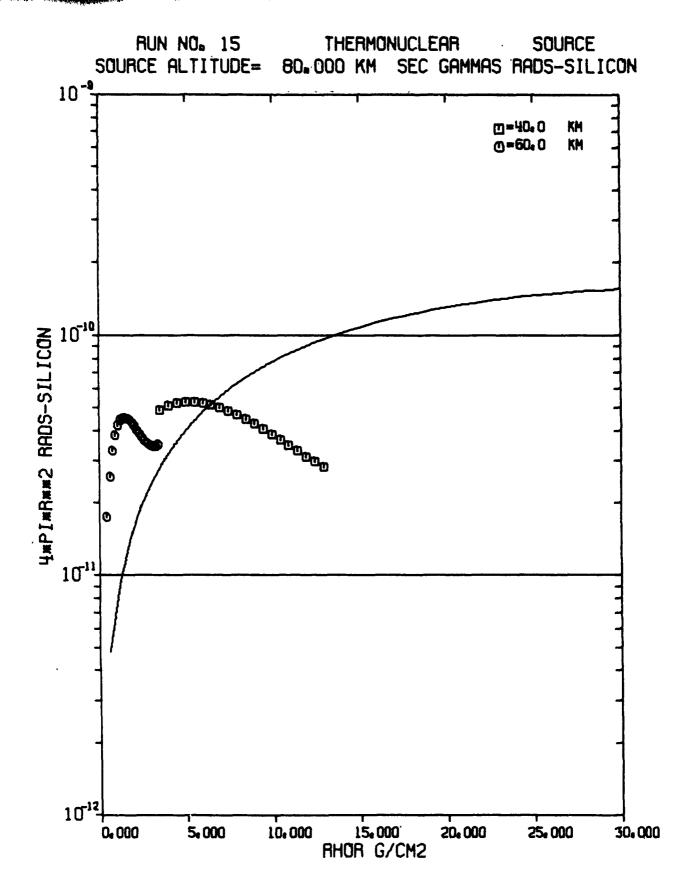


FIGURE C-/9 MORSAIR FIT DATA-4PIR**2 GAMMA SI DOSE.
THERMONUCL SOURCE IN REAL AIR AT 80.0 KM.
SAMPLING ALTITUDES 40 AND 60 KM.

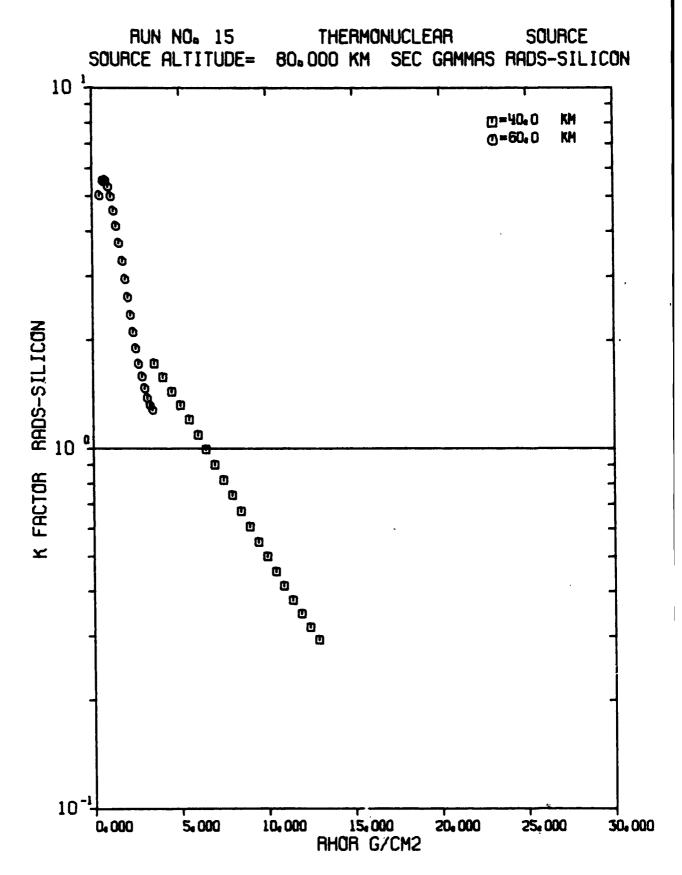


FIGURE C-00 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR. THERMONUCL SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 40 AND 60 KM.

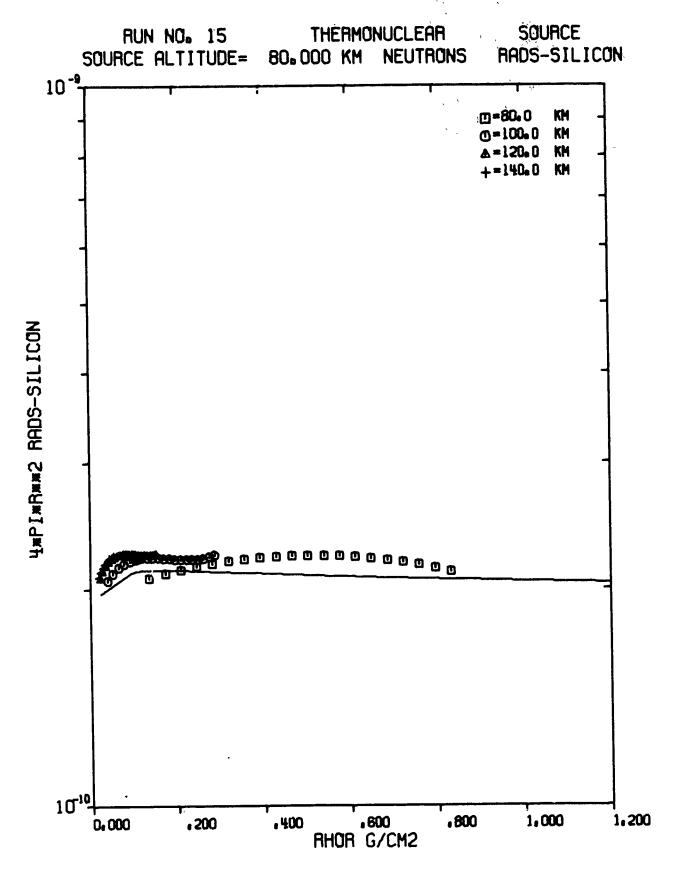


FIGURE C-81 MORSAIR FIT DATA-4PIR**2 NEUTRON SI DOSE, THERMONUCL SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 80,100,120, AND 140 KM.

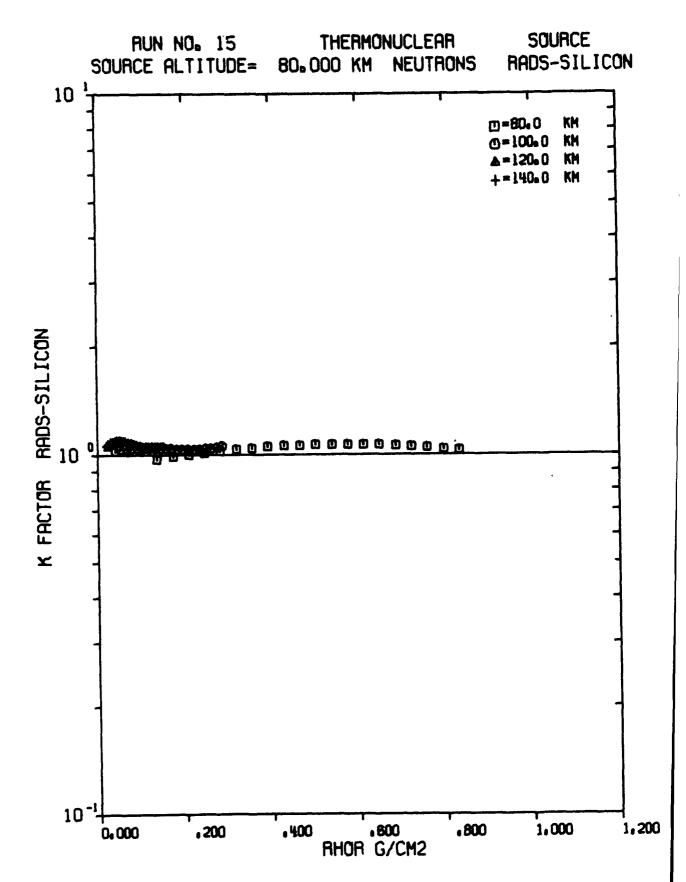


FIGURE C-82 MORSAIR FIT DATA-NEUTRON SILICON K-FACTOR THERMONUCL SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 80,100,120, AND 140 KM.

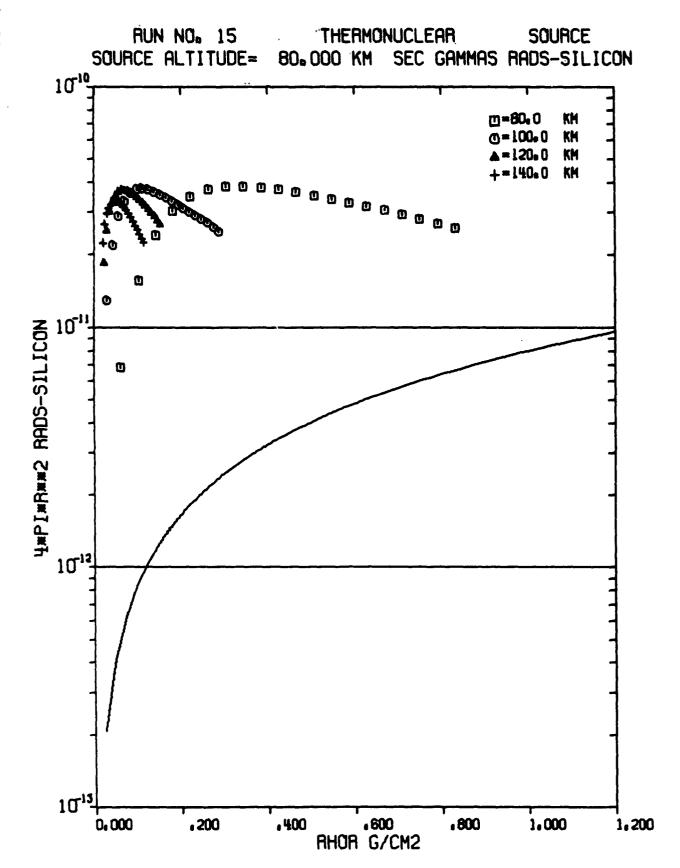


FIGURE C-83 MORSAIR FIT DATA-4PIR**2 GAMMA SI DOSE, THERMONUCL SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 80,100,120, AND 140 KM.

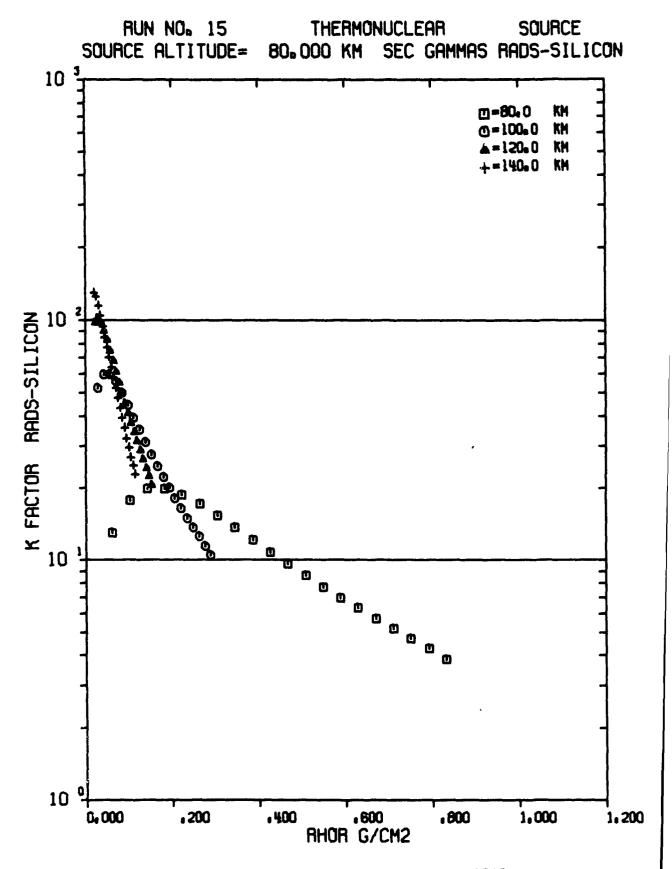


FIGURE C-84 MORSAIR FIT DATA-GAMMA SILICON K-FACTOR, THERMONUCL SOURCE IN REAL AIR AT 80.0 KM. SAMPLING ALTITUDES 80,100,120, AND 140 KM.